

18 May, 2022

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Standardized Irradiation Testing and Multi-Modal Characterization for Advanced Nuclear Materials

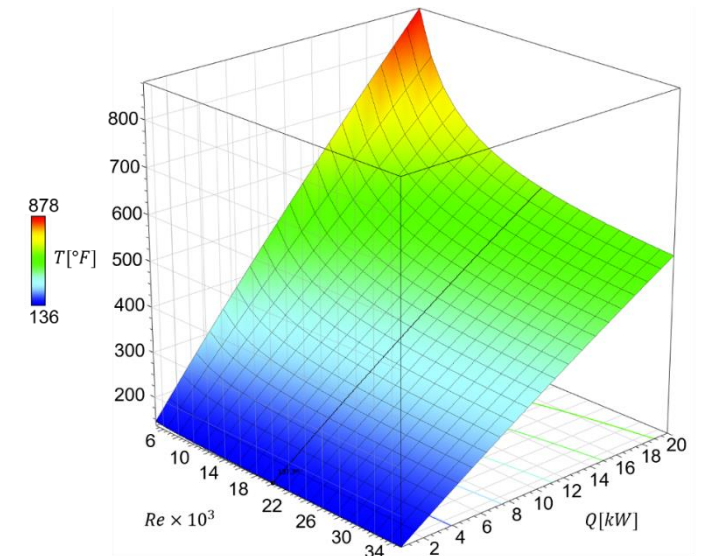
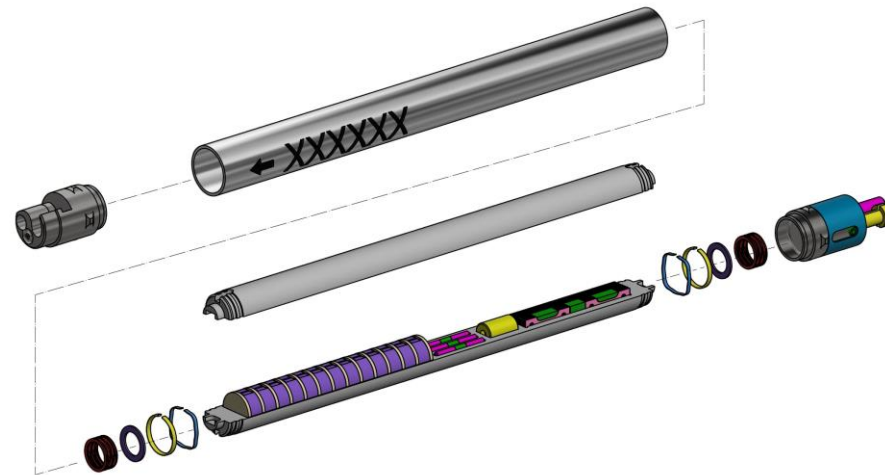
INL Testing and Characterization Efforts

- Development of standard irradiation capsule ISHA-1
 - Using for SS316H and 16-8-2 Weld Metal
- In-Situ Property Measurement
 - Demonstration of single head thermal reflectance stage within G4 Hydra
 - Development of additional modalities using the feedthrough
- ANCERS, Application of Non-destructive Combinatorial Examination of Radioactive Samples
 - Design of Gamma-Ray Emission Tomography Assay (GRETA) table
 - Developing reconstruction methods using standard samples
- High temperature alloy characterization

FY21 - Universal Drop-in Capsule for ATR Testing

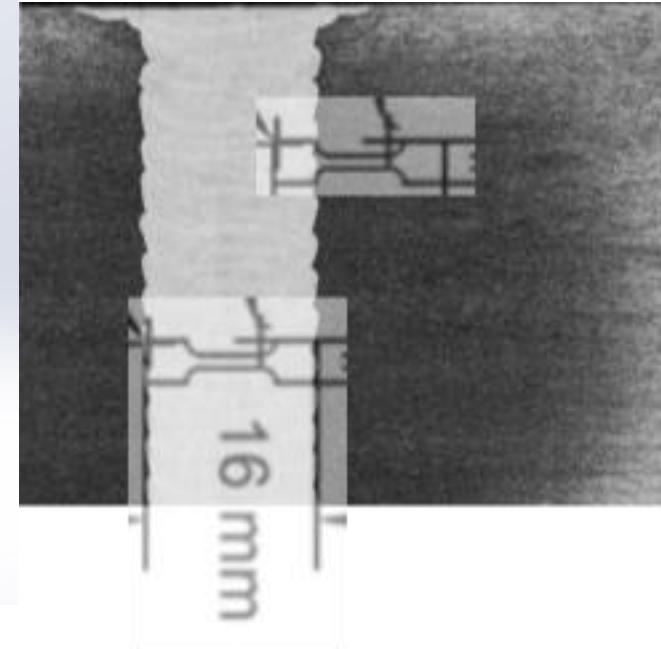
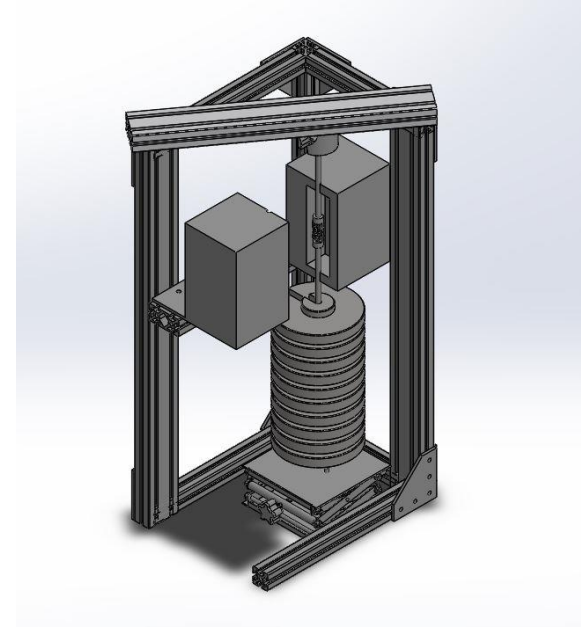
- Under NMDQi, we developed a semi-universal drop-in capsule for irradiation in the Advanced Test Reactor (ATR) called the Irradiation System for High-throughput Acquisition (ISHA-1). This utilizes a standard outer capsule that can facilitate both fuel experiments or structural material specimens in a variety of ATR Positions.
 - Structural material testing can support compact tension, bend bar, and tensile specimen geometries for temperatures up to 800 °C.
 - Utilizes concentric ASME pressure boundaries in order to support more reactive material irradiation testing within ATR, such as molten salts and other reactive materials.

ISHA enables a rapid design cycle for irradiation testing that provides significant savings and standardized specimen geometries



SS316 Irradiation Test

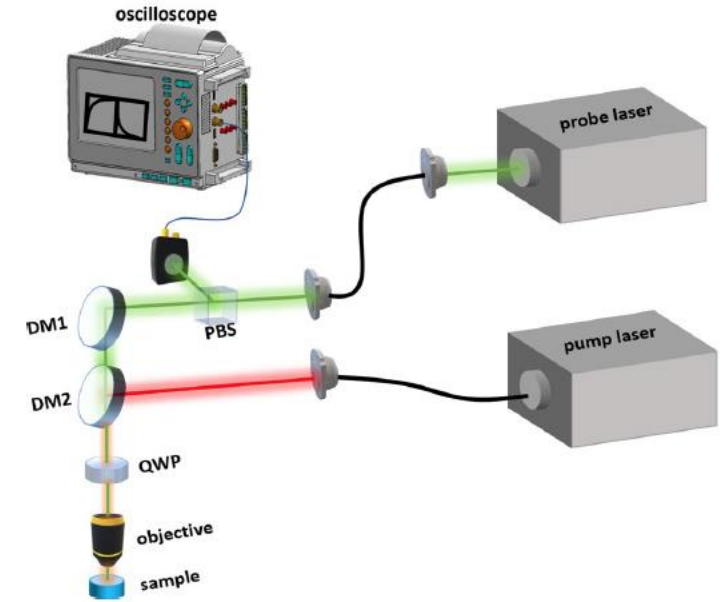
- Partnership with Kairos Power ARDP award and NMDQi efforts
 - Irradiating welded SS316H with AWS 16-8-2 weld metal
 - Temperatures of 550 °C and 650 °C
 - Target doses of <1 dpa, 1-2 dpa, 10-15 dpa
- Objectives will be to mechanically characterize difference between the base metal, weld metal, and heat affected zone
- National Reactor Innovation Center (NRIC) is funding the design, assembly, and deployment of three in-cell load frames to perform the mechanical tests
- Multiple creep rupture lifetimes being investigated
- FY22-FY27 effort
 - Irradiation and PIE funded through ARDP award



High temperature creep testing of 316 and small scale specimens

Thermal Reflectance Method and the SPTR

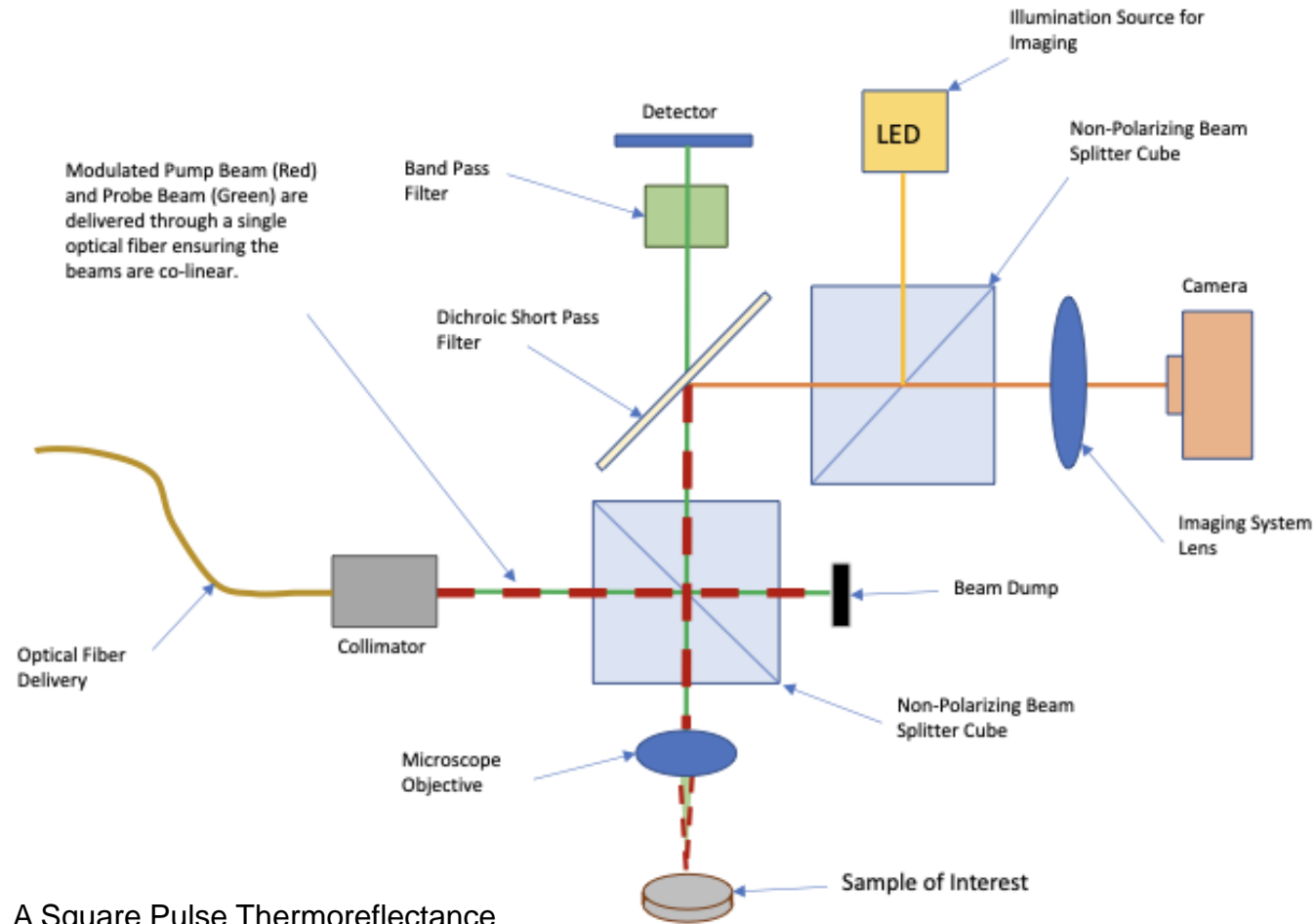
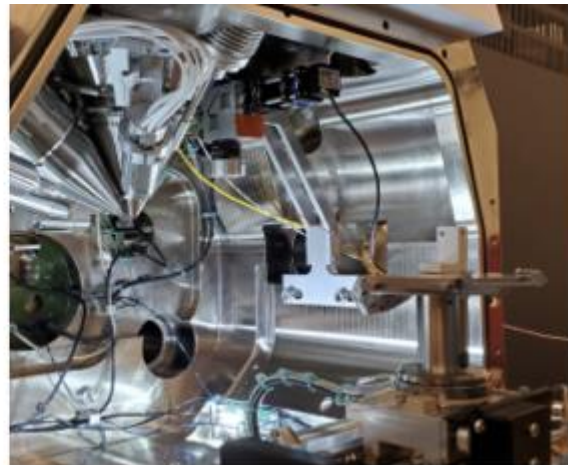
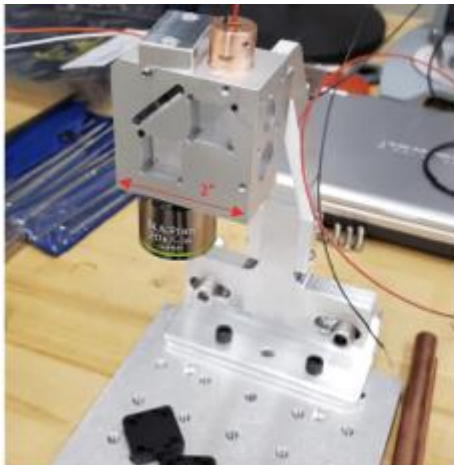
- Thermal reflectance methods utilize the small, local change in optical reflectance of a material based upon its temperature
 - Can compare temperature vs frequency (frequency domain) or temperature vs time (time domain)
 - SPTR utilizes a time domain interpretation
- Thermal reflectance is the basis for the Thermal Conductivity Microscope (TCM) at the Irradiated Materials Characterization Laboratory (IMCL)
- The square pulse thermal reflectance (SPTR) method was developed to support in-situ testing within the G4 Hydra PFIB/SEM
 - Spot sizes on the order of 2 μm on the specimen surface



**Explicitly correlating
thermal/mechanical
properties to
microstructural features
during PIE**

Design of Thermal Reflectance Head in G4 Hydra

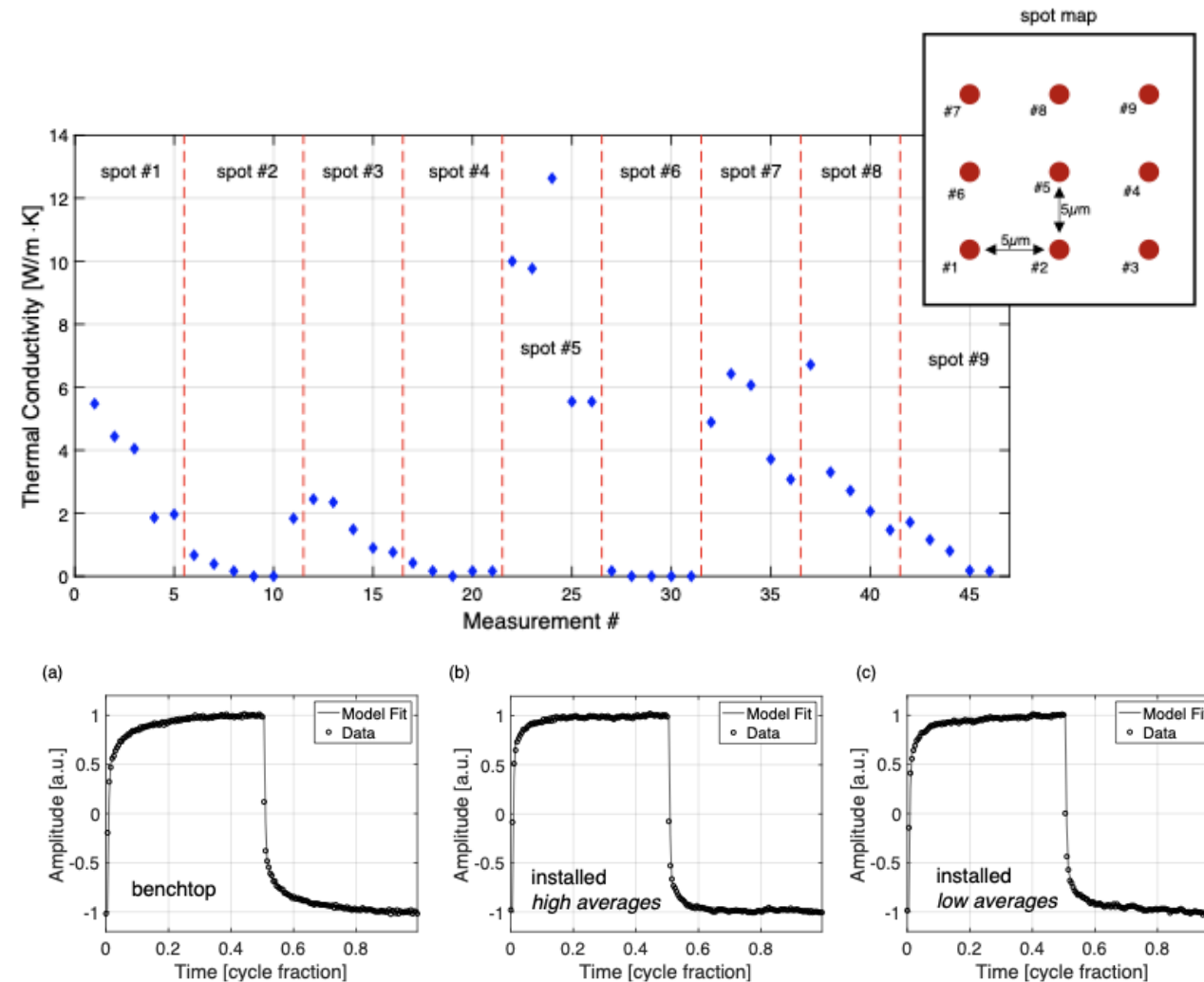
- In-Situ testing property testing during microscopy
- Stage compresses both the probe and pump lasers into a single optic that can support laser change outs from outside the SEM
- Feed through a vacuum port on the SEM chamber



A Square Pulse Thermoreflectance Technique for the Measurement of Thermal Properties
Y Wang, V Chauhan, Z Hua, R Schley, CA Dennett... - International Journal of Thermophysics, 2022

Square Pulse Thermal Reflectance (SPTR) Results

- Investigation of the method focused on identifying sensitivities and proving the model
 - Spot size, film (transducer) size, and model variables
- Emphasis was placed on relationships between material conductivity and film thickness
- Validation with BK7 glass and gold films
 - Benchtop was within 3% of known values for BK7 (~ 1 W/m/K)
 - Installed values were initially not good ($\sim 3-4$ W/m/K)

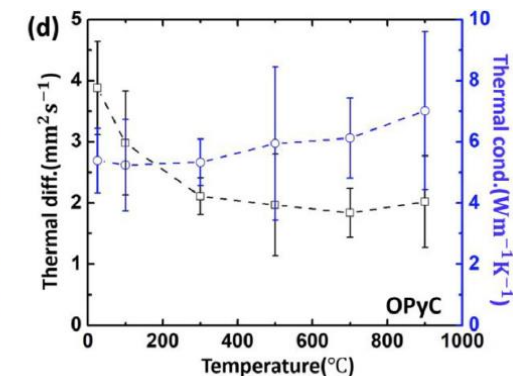
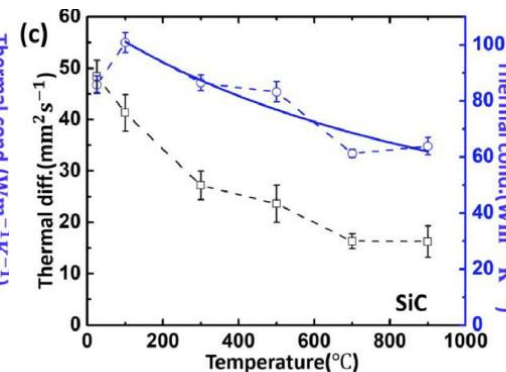
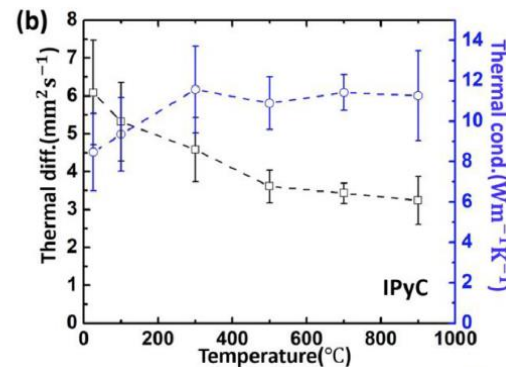
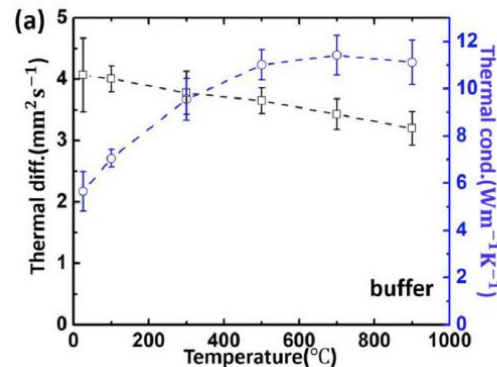
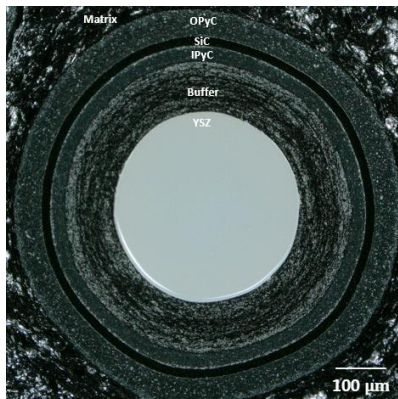


Investigation of TRISO Particle in G4

- A tri-isotrostructure (TRISO) fuel particle was used as a basis for demonstrating the SPTR instrument within the G4
 - Well established material properties for comparison
 - Small, well defined layers to distinguish the results from
- Results were also compared to a similar instrument, the TCM at IMCL

		SPTR (W/mK)*	TCM	
			Therm cond.(W/mK)	Therm diff.(mm ² /s)
RT	YSZ	2.22	2.65	0.9
	Buffer	3.64	3.25	3.25
	IPyC	4.75	5.16	5.56
	SiC	74.1	84.3	38.3
	OPyC	4.45	3.96	3.52

*assume heat capacity from literature

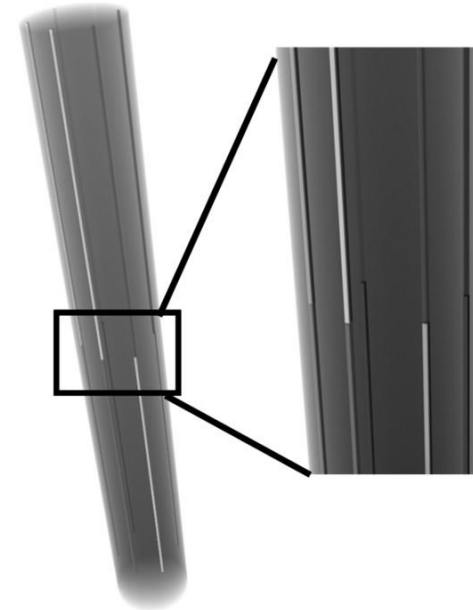


Thermal properties measurement of TRISO particle coatings from room temperature to 900 °C using laser-based thermoreflectance methods

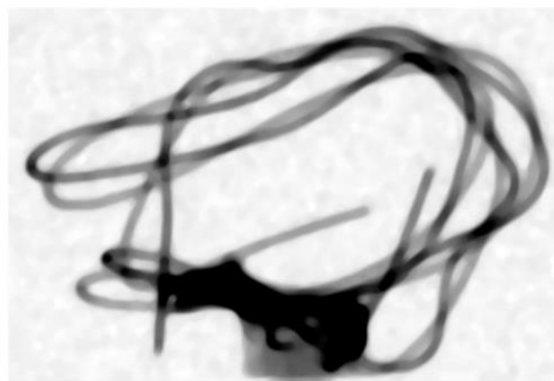
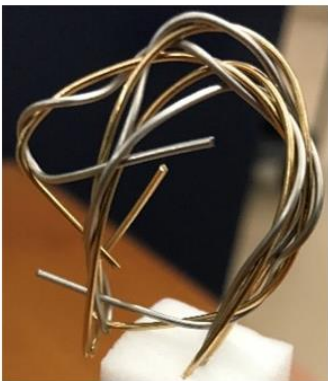
Y Wang, Z Hua, R Schley, G Beausoleil II, DH Hurley - Journal of Nuclear Materials, 2022

FY21: Began Development of Integrated Tomographic Methods for Characterization for Irradiated Materials

- To increase the throughput of materials testing and characterization, combinatorial tomographic methods are under development with the goal of increasing the scientific impact of analysis but combining the data output of a single sample characterized with multiple investigative methods.
 - Application of Non-destructive Combinatorial Examination of Radioactive Samples (ANCERS)
 - Provide isotopic and spatial resolution $<10\ \mu\text{m}$ on large, dense irradiated specimens

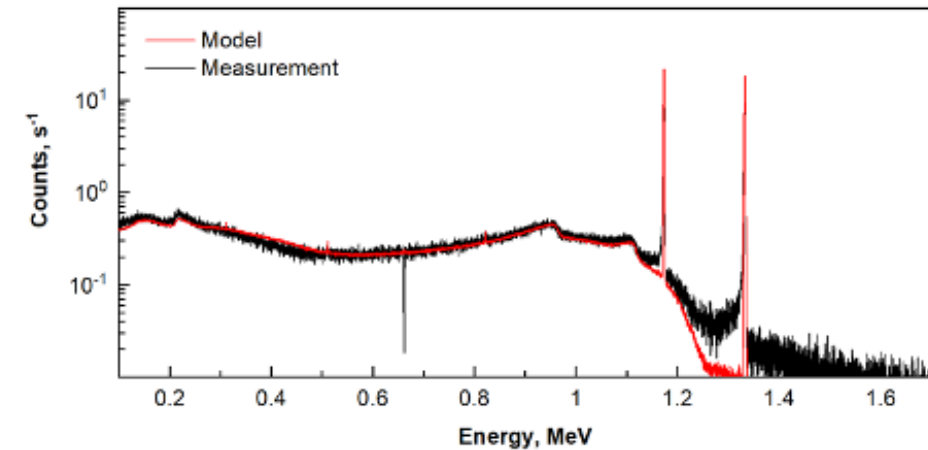
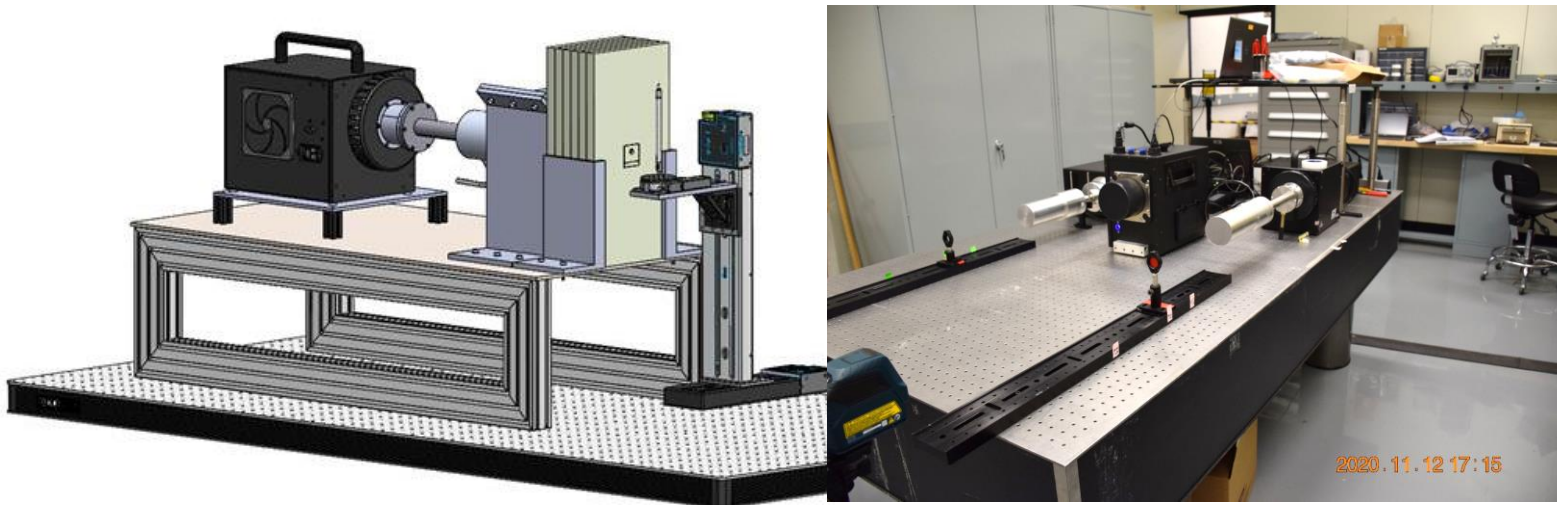
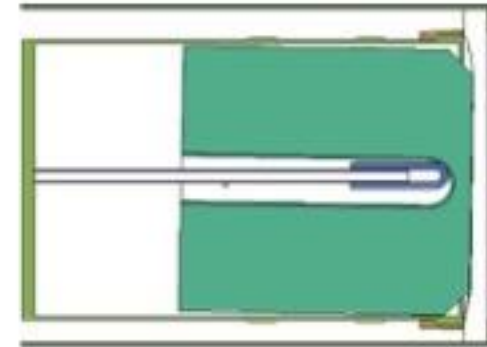


Combining tomographic methods allows each to make up for the others' shortcomings, thus improving the value proposition of tomography on irradiated materials



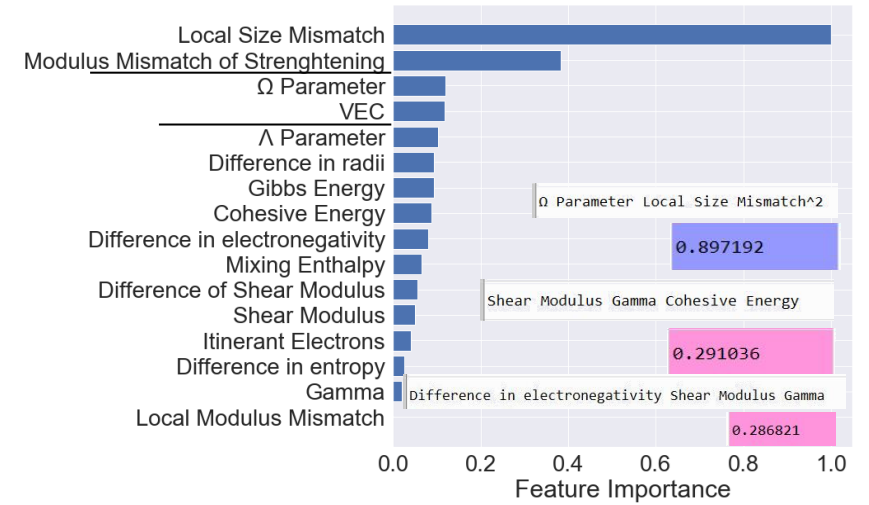
Gamma Ray Emission Tomography Assay (GRETA) System

- Gamma emission tomography (GET) instrument to support ANCERS
- A high purity Ge detector was received and characterized
- Unfortunately, there were some problems with the received detector and the vendor is in process of replacing it.

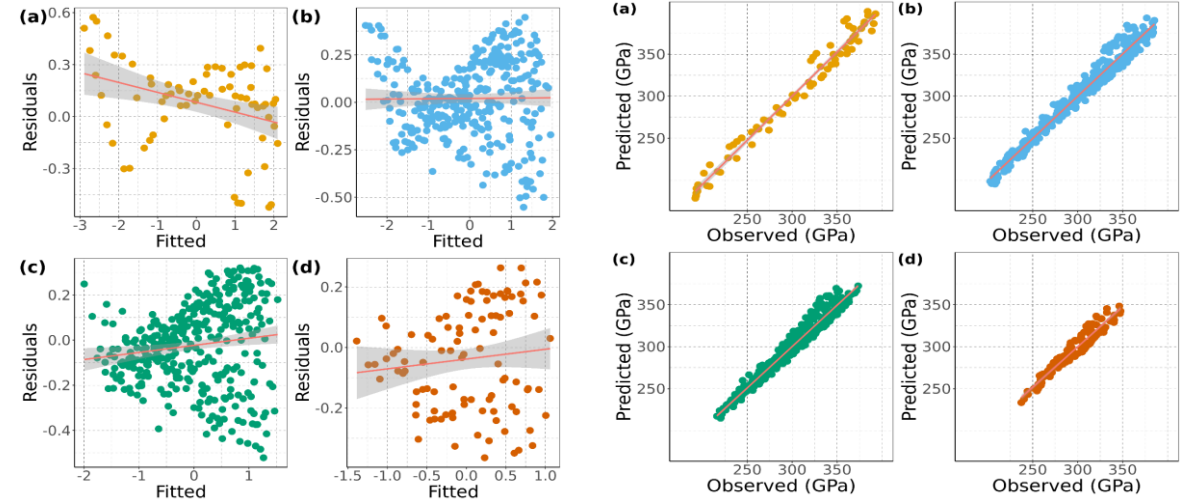


High Temperature Material Development

- Work initiated from NMDQI
- Objective was to continue investigations on materials developed from an LDRD at INL for high temperature reactor core applications including structural materials and potential cladding
- Focus on multi-principal element alloys (MPEAs)



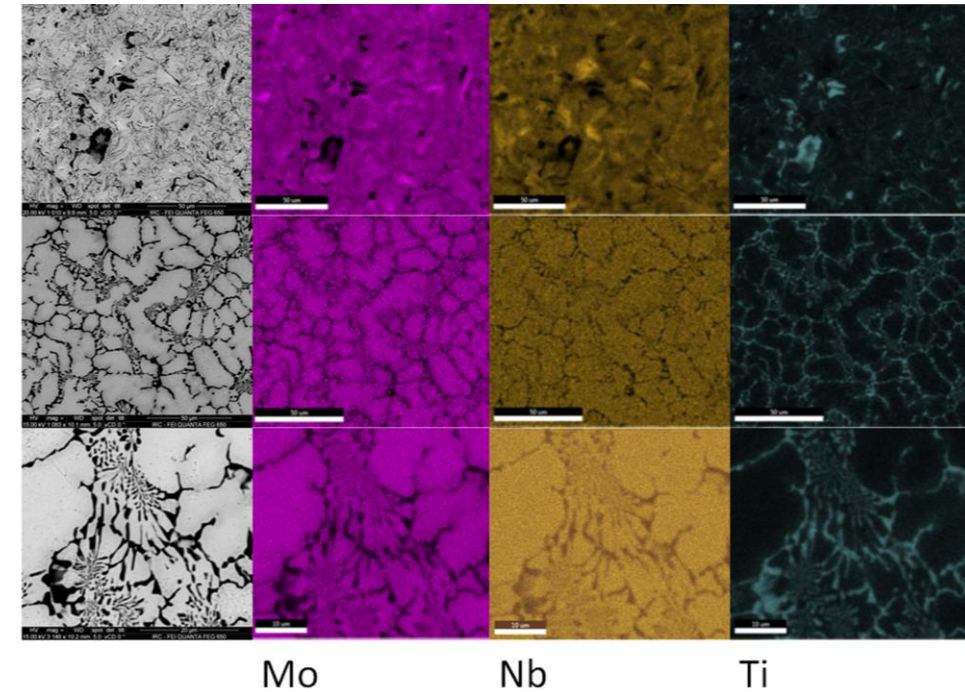
A group						D group							
Iron 26 Fe 55.845	Chromium 24 Cr 51.996	Nickel 28 Ni 58.693	Manganese 25 Mn 54.938	Copper 29 Cu 63.546	Aluminum 13 Al 26.982	Silicon 14 Si 28.085	Iron 26 Fe 55.845	Chromium 24 Cr 51.996	Nickel 28 Ni 58.693	Cobalt 27 Co 58.933	Manganese 25 Mn 54.938	Copper 29 Cu 63.546	
B group						BB group							
Iron 26 Fe 55.845	Chromium 24 Cr 51.996	Nickel 28 Ni 58.693	Manganese 25 Mn 54.938	Copper 29 Cu 63.546	Aluminum 13 Al 26.982	Cobalt 27 Co 58.933	Silicon 14 Si 28.085	Iron 26 Fe 55.845	Chromium 24 Cr 51.996	Aluminum 13 Al 26.982	Nickel 28 Ni 58.693	Manganese 25 Mn 54.938	Carbon 6 C 12.011
C group						E group							
Iron 26 Fe 55.845	Manganese 25 Mn 54.938	Carbon 6 C 12.011	Aluminum 13 Al 26.982	Silicon 14 Si 28.085	Chromium 24 Cr 51.996	Chromium 24 Cr 51.996	Vanadium 23 V 50.942	Tantalum 73 Ta 180.95	Tungsten 74 W 183.84	Iron 26 Fe 55.845	Manganese 25 Mn 54.938	Chromium 24 Cr 51.996	Titanium 22 Ti 47.867
F group						G group							
Iron 26 Fe 55.845	Manganese 25 Mn 54.938	Molybdenum 42 Mo 95.95	Silicon 14 Si 28.085	Carbon 6 C 12.011	Chromium 24 Cr 51.996	Iron 26 Fe 55.845	Chromium 24 Cr 51.996	Vanadium 23 V 50.942	Tantalum 73 Ta 180.95	Iron 26 Fe 55.845	Manganese 25 Mn 54.938	Chromium 24 Cr 51.996	Titanium 22 Ti 47.867
H group						I group							
Iron 26 Fe 55.845	Chromium 24 Cr 51.996	Nickel 28 Ni 58.693	Manganese 25 Mn 54.938	Silicon 14 Si 28.085	Carbon 6 C 12.011	Iron 26 Fe 55.845	Chromium 24 Cr 51.996	Vanadium 23 V 50.942	Tantalum 73 Ta 180.95	Iron 26 Fe 55.845	Chromium 24 Cr 51.996	Nickel 28 Ni 58.693	Copper 29 Cu 63.546
J group						K group							
Iron 26 Fe 55.845	Chromium 24 Cr 51.996	Nickel 28 Ni 58.693	Manganese 25 Mn 54.938	Silicon 14 Si 28.085	Carbon 6 C 12.011	Iron 26 Fe 55.845	Chromium 24 Cr 51.996	Vanadium 23 V 50.942	Tantalum 73 Ta 180.95	Iron 26 Fe 55.845	Chromium 24 Cr 51.996	Nickel 28 Ni 58.693	Copper 29 Cu 63.546



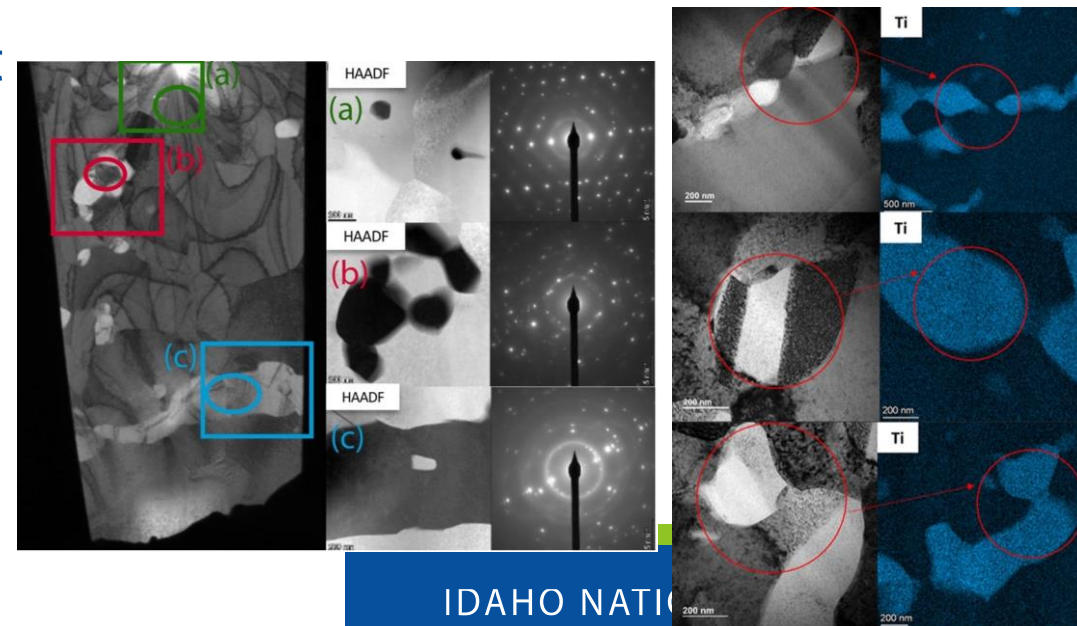
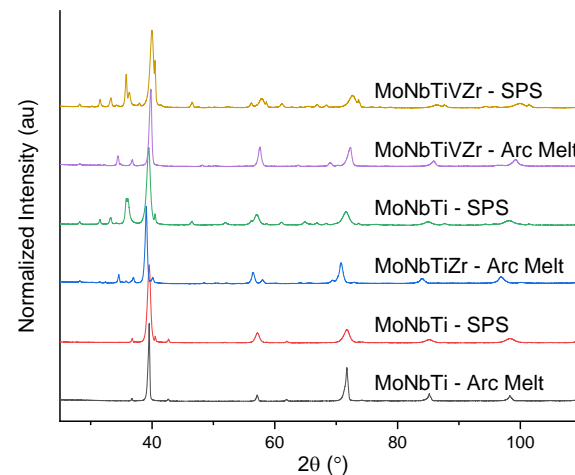
M. Grant, R. Kunz, K. Iyer, L. Held, T. Tasdizen, J. A. Aguiar, and P. P. Dholabhai, "Combining Atomistic Simulations and Machine Learning to Design Multi-principal Element Alloys with Superior Elastic Modulus"

Alloy Analysis

- Investigating high temperature stability and phases of MoNbTi based MPEAs
 - Fabricated using cryogenic milling and SPS
 - MoNbTi mixed with Zr, VZr, and CrV
 - Comparisons with arc-melt methods
- Current work
 - Alloys currently undergoing tensile testing
 - TTUSC irradiation test awaiting ATR re-start



Validating computational models used to design high temperature alloys

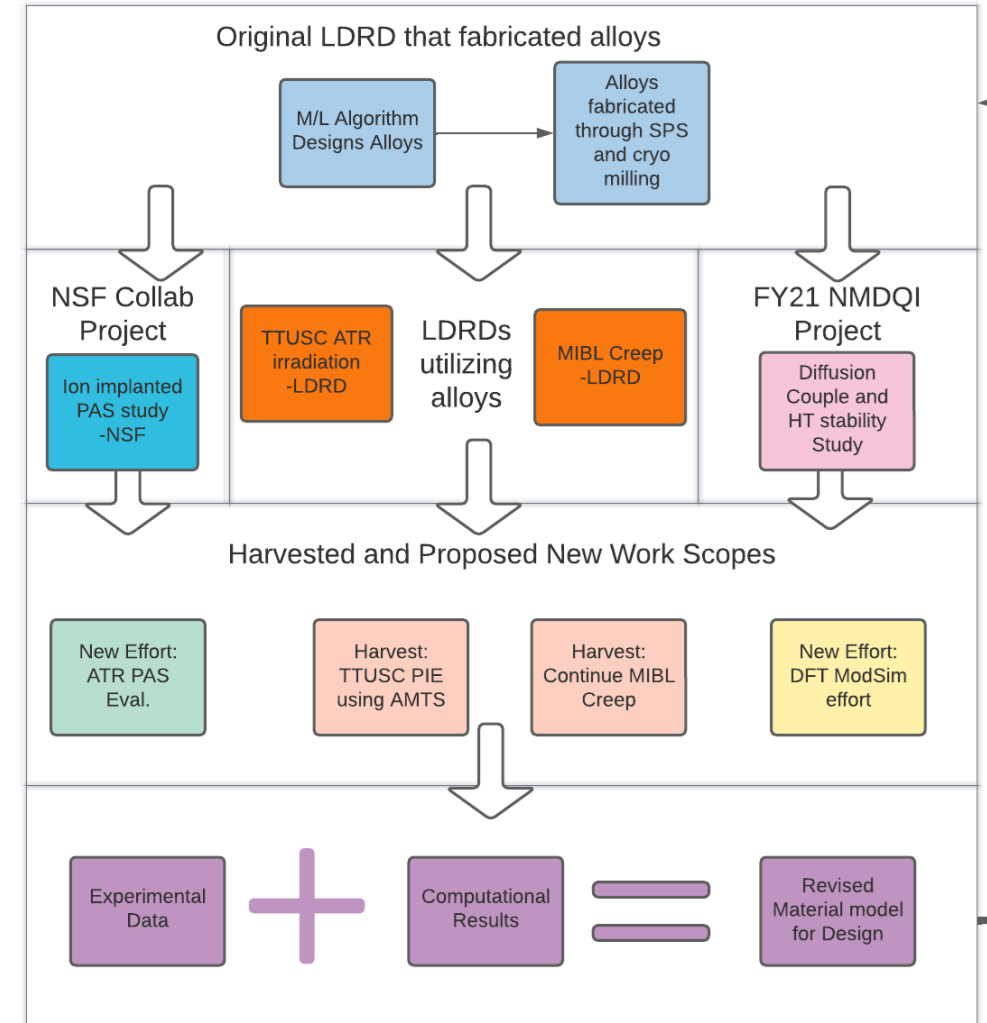




Idaho National Laboratory

FY23-FY25 High Temp Strategy

- Advanced Alloy Design
 - Machine learning algorithm to predict material mechanical properties
 - Cryogenic milled powders and SPS to fabricate alloys
 - Phase and mechanical testing
- High temperature phase stability and property assessment
- TTUSC – Neutron irradiation and mechanical testing
- MIBL Creep Rig – in-situ ion irradiation and creep testing
- NSF Positron Annihilation Spectrometry
- Addition of ModSim development for atomistic behaviors



Demonstrates
experimental framework
for testing new alloys

Utilizing ATF-2 Irradiation Opportunities

- Vendors often miss insertion dates and are replaced by dummy capsules
 - Conditions are within a flux trap typical of PWR core
 - 300 series SS
 - Simple cylinder for analysis credit
 - Typical minimum irradiation time of 2-3 cycles in the position
- The source of these cylinders is somewhat irrelevant and so the option of fabricating A/M SS316 cylinder
 - Fabricate a collection of A/M SS316 capsules with varying parameters and keep on hand to perform irradiations when available

Essentially a 'free' ATR
irradiation in PWR Conditions

