

(Microreactor Applications Research, Validation & Evaluation),

MARVEL Dry Critical: Fuel Loading and Initial Startup Neutronics Analysis

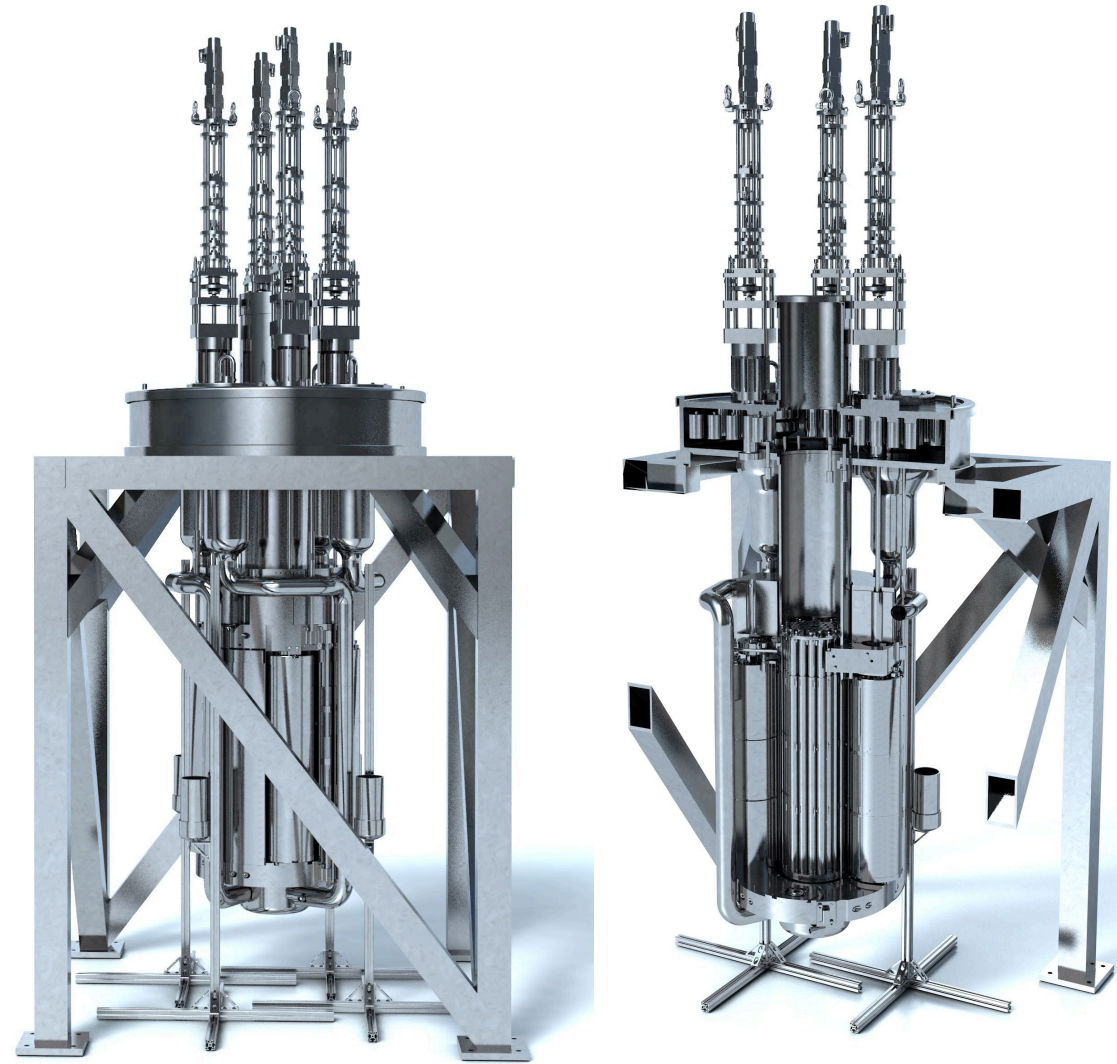
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INL/MIS-26-90673

MARVEL Dry Crit Overview

- **MOTIVATION**
 - Provide de-risking of key characteristics of MARVEL design
 - Validation of reactor physics models for excess reactivity and shutdown margin
 - Create a neutronic benchmark for industry, researchers, academia
- **CHARACTERISTICS:**
 - Ability to decouple from other project efforts (e.g., TREXC pit)
 - Ability to make adjustments/repairs as needed (Guard Vessel welding is irreversible)
 - Ability to validate neutronics sooner (similar to PCAT goals)
- **SCOPE:**
 - **Included:** PCS, Fuel, Reflector, RCS, NI
 - **Not included:** Insulation, TCs (installed but not connected), GVS installation, T-REXC pit completion (experiment can be conducted on TREAT floor), HES, PGS

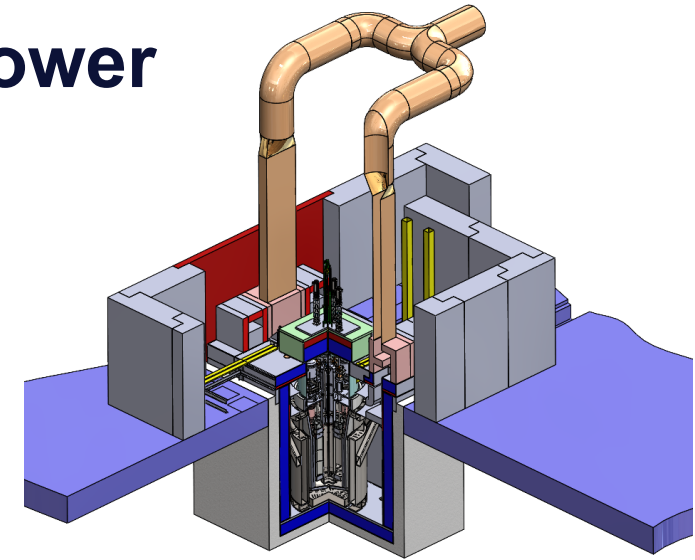


Key Hardware Tests Along The Path To Full Power

**PCAT Hydraulic
Validation Testing**
Complete: *Summer 2025*

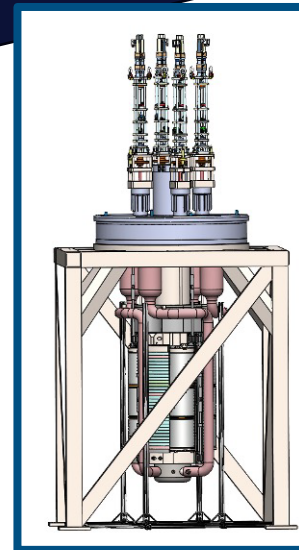


**MARVEL Reactivity Control
Qualification**
Target: *Fall 2025*



1. Reactor install in pit
 2. HES install
 3. T-REXC completion
 4. NaK Loading
 5. Zero Power Physics Test
 6. Full Power Test
 7. Turnover to Ops
- Target: *Summer 2028*

**MACS Startup Procedure
Testing**
Complete: *Summer 2025*



Proposed modified
configuration:

MARVEL Dry Criticality Test
Target: *Winter 2026 – Spring 2027*

MARVEL Initial Criticality – Dry

PLN-25056

- No Shielding
- No Guard Vessel
- No CIA Assembly
- On TREAT floor (not in pit)
- From T-REXC:
 - I&C
 - Radiation Area Monitors
 - AC Power Distribution
- 30-34 Fuel Pins depending on as-built
 - Load to $k_{\text{eff}} \leq 0.9$ (ANS/ANSI-1)
- Operate from TREAT Control Room
 - Incremental Drum Withdrawal to criticality
 - Use of control drum hardstops, if necessary

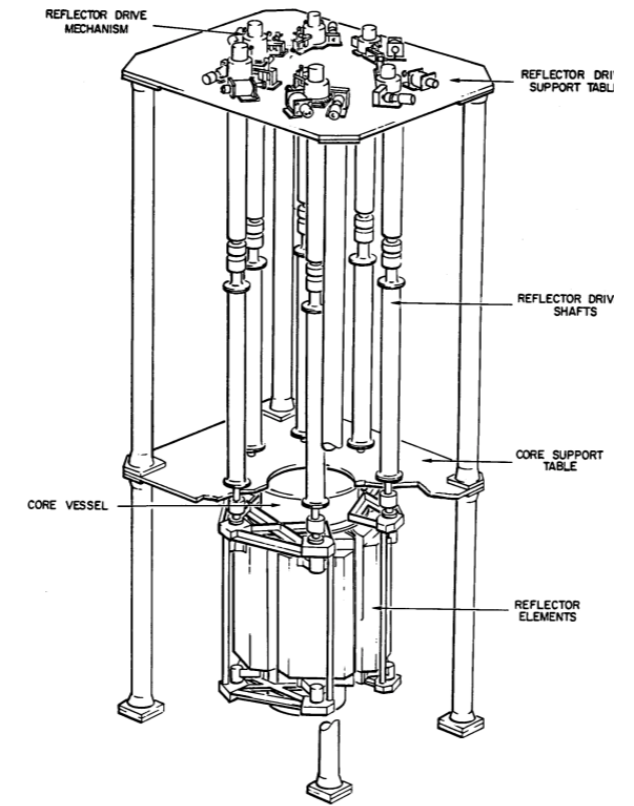
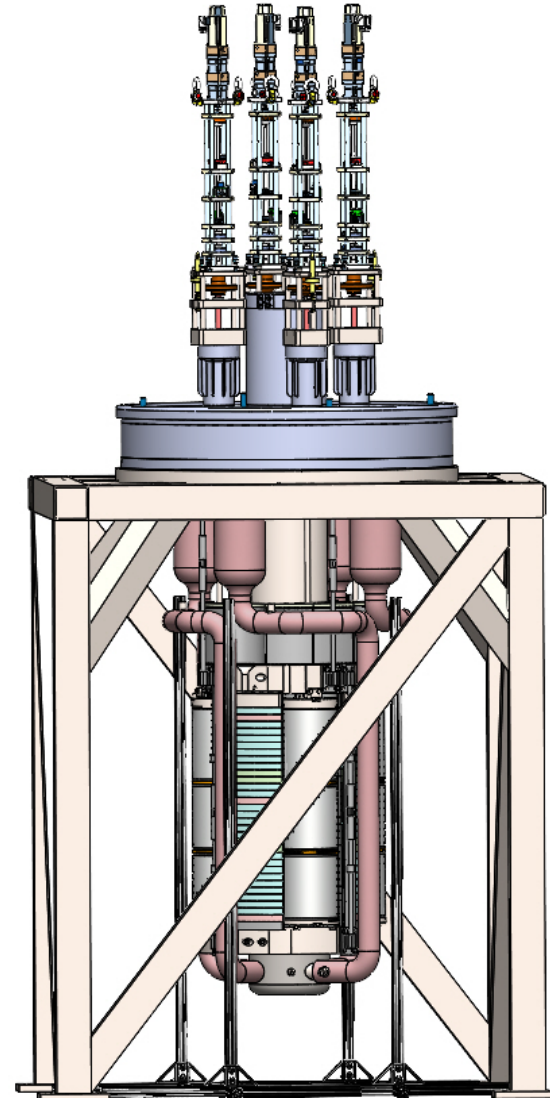


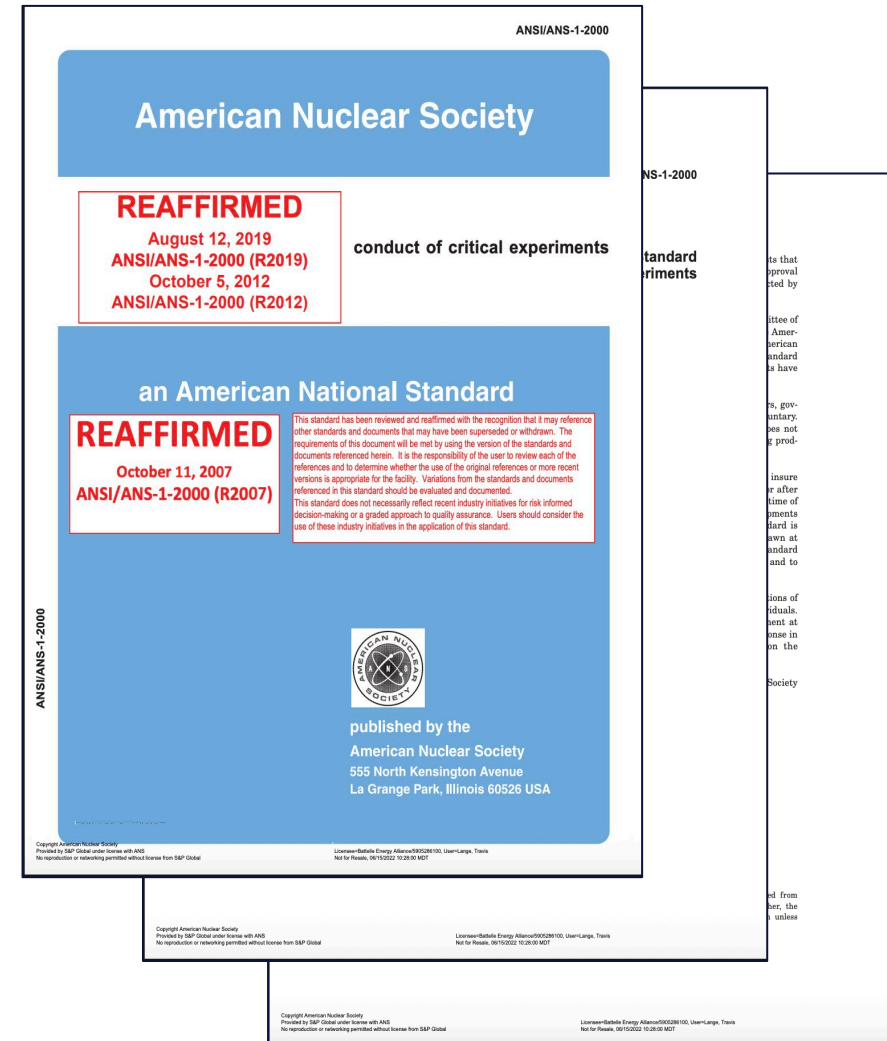
Figure 2. Critical Machine

SNAP 8 Critical Experiment

Conduct of Critical Experiments as Guidance

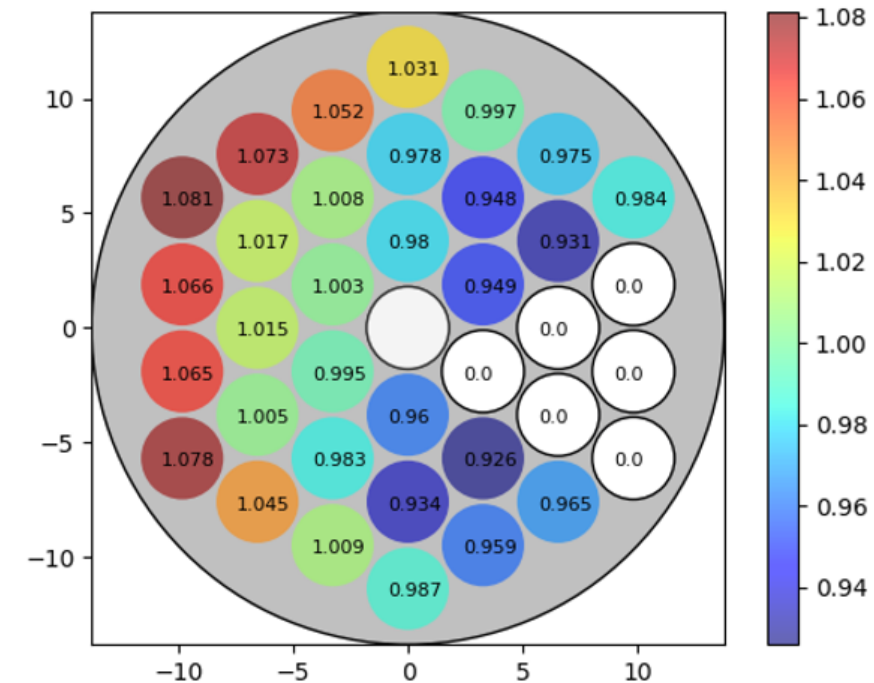
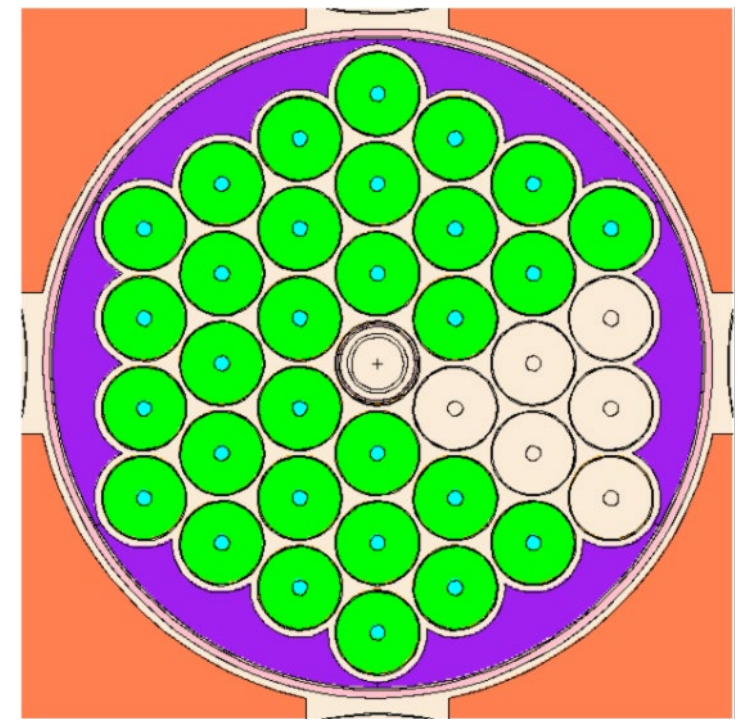
• ANSI/ANS-1-2000 being used for guidance on fuel loading and initial startup:

- Program of experiments documented, independently reviewed, and approved (sec. 3.3)
- Predicted $k_{\text{eff}} < 0.9$ for manual loading of “unknown configurations” (sec. 3.9)
- Predicted $k_{\text{eff}} < 0.95$ for manual loading of “known configurations” (sec. 3.9)
- Use of startup neutron source (sec. 4.4)
- Nuclear instrumentation (sec. 4.9)



Neutronics Analysis for Partial Fuel Loading

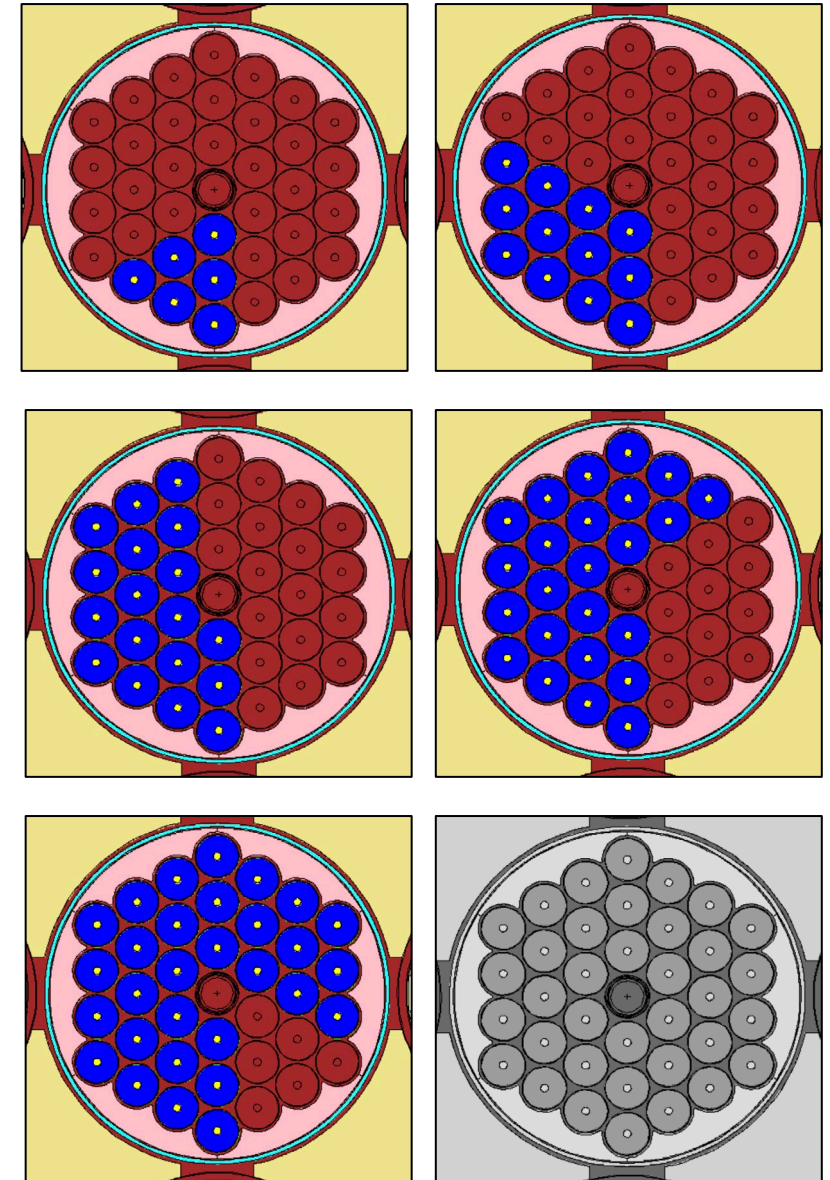
- Calculated that 30 fuel rods (5 six-packs of fuel) meet conditions
 - $k_{\text{eff}} \leq 0.9$ for fuel loading with drums fully inserted
 - Worth of control drums $> 10\%$
 - Can be made critical ($k_{\text{eff}} = 1.0$) with control drums operated remotely
- Safety basis analyses completed
 - 2D power peaking factors
 - Integral control drum worth curves
 - Kinetics parameters



Fuel Loading Sequence Neutronics Analysis

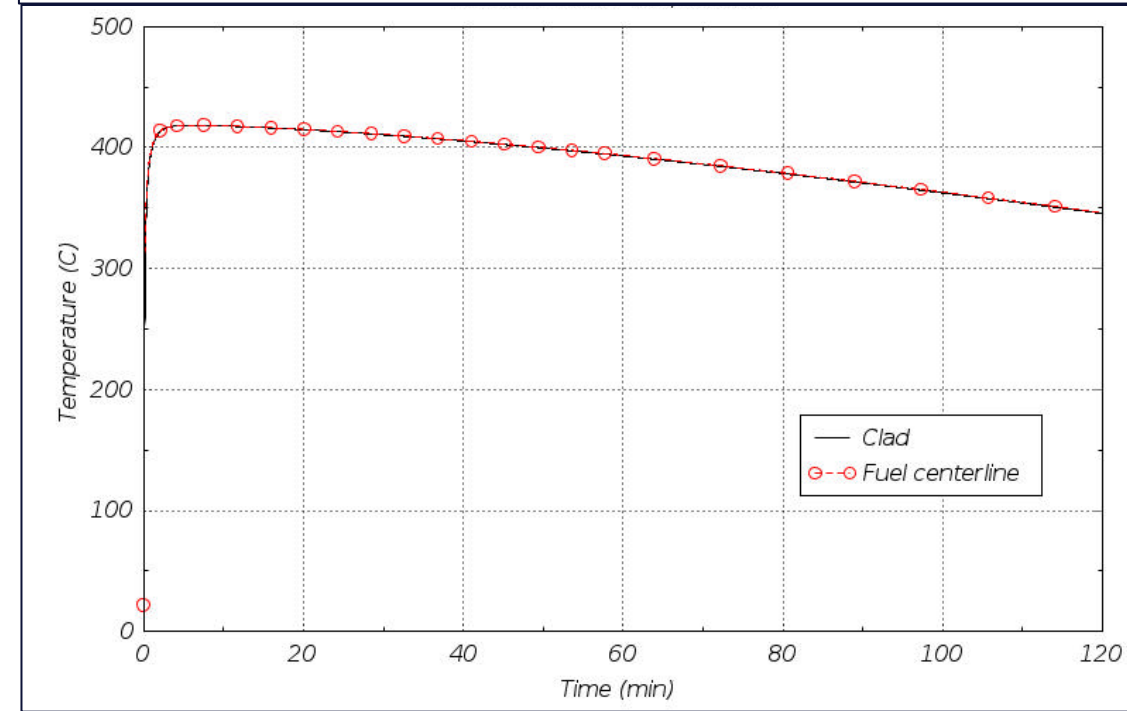
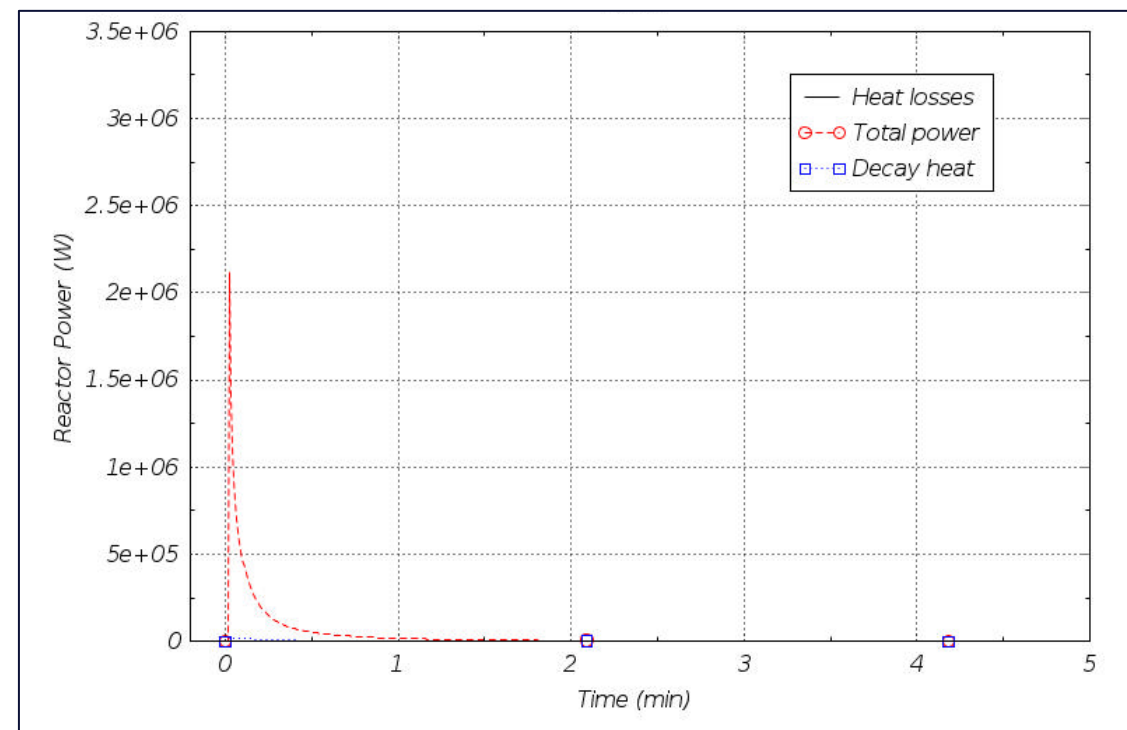
NUMBER OF SUBASSEMBLIES	THERMAL FLUX AT DETECTOR [N/CM2/S]	K-EFFECTIVE	CALCULATED COUNTS PER SECOND AT DETECTOR
0	14.1	0.00000	56.4
1	13.2	0.30832	52.8
2	13.5	0.53343	54.0
3	14.5	0.65979	58.0
4	16.0	0.75186	64.0
5	19.2	0.84106	76.8
6*	29.4	0.92859	117.6

* Full core loading not used for initial approach to criticality. Available as additional experimental configuration after initial criticality with partial fuel loading.

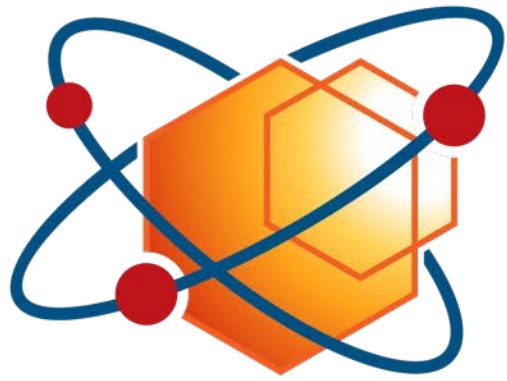


Updating RELAP Dry RIA Transient

- Unprotected Transient Overpower (UTOP) at DRY-CRIT conditions simulated
- Update of the MARVEL RELAP5-3D model
 - Hot Channel Factor (HCF)
 - Core flow area
 - Neutronic parameters
- Simulation of 1.33\$ prompt reactivity insertion
- Fuel and clad temperature peaking at ~ 410 °C
- Safety margins OK for clad and fuel
 - Conservative hot channel factor, conservative clad safety limit



Questions?



MRP Microreactor
Program