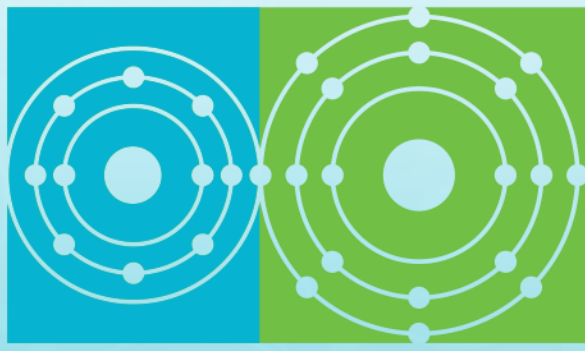


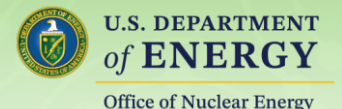


Molten Salt Reactor
P R O G R A M

Overview of the Molten Salt Reactor Program

Dr. Patricia Paviet, National Technical Director

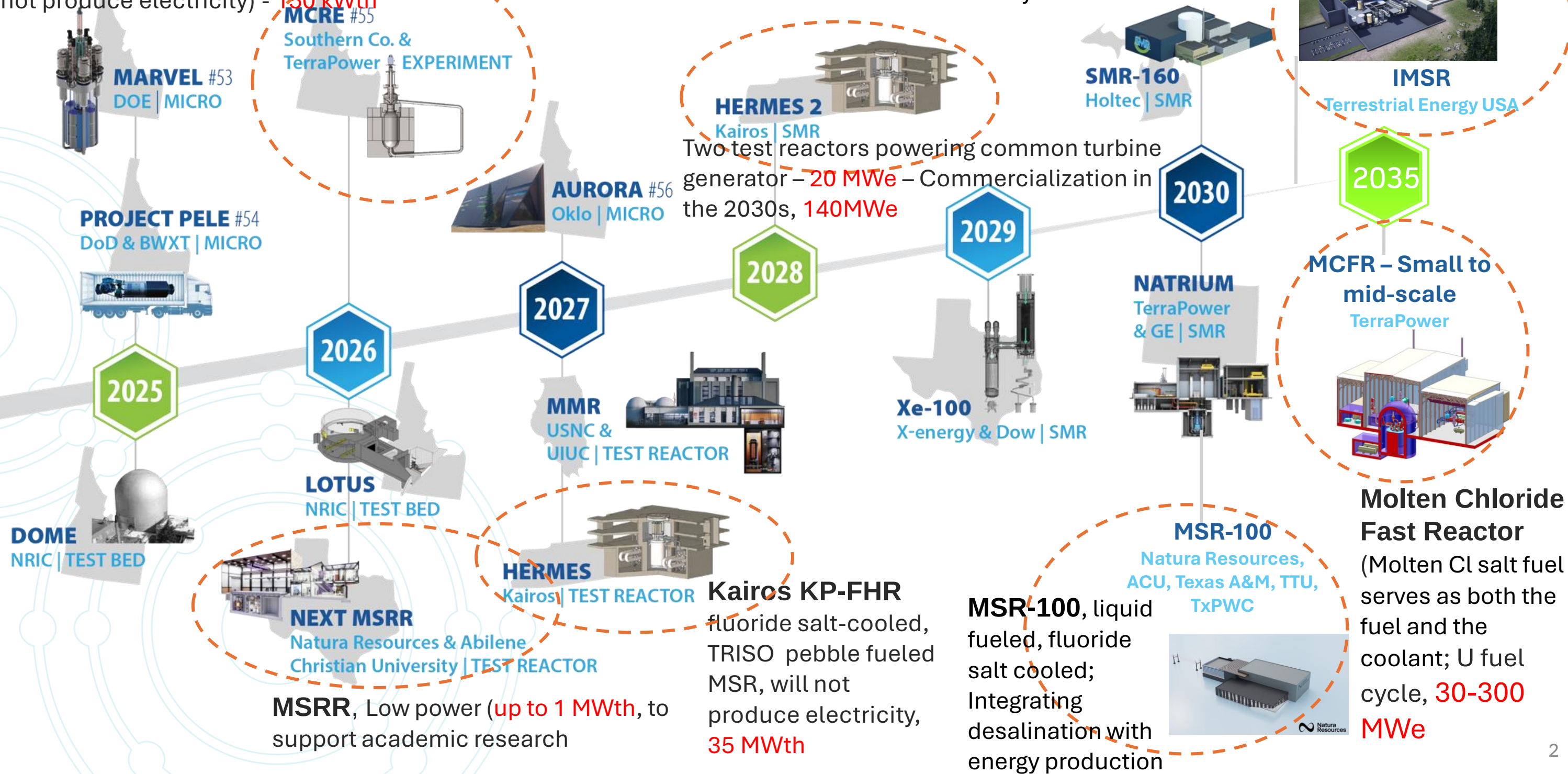
Annual MSR Campaign Review Meeting 22-24 April 2025



Accelerating Advanced Reactors Demonstration & Deployment in the U.S.

Molten Chloride Reactor Experiment (only liquid fueled design supported under ARDP – will not produce electricity) - **150 kWth**

Integrated Molten Salt Reactor (molten fluoride salt reactor; **390-MWe** Generation IV UF₄ (LEU) fuel in a molten fluoride salt system.



Kairos Power Starts Construction of Hermes Reactor

Kairos Power has started construction on one of the first advanced reactors in the United States.

July 30, 2024



Kairos Power

30,642 followers

20h • Edited •

The [Office of Nuclear Energy | U.S. Department of Energy](#) has conditionally agreed to commit High-Assay Low-Enriched Uranium via the HALEU Availability Program for the ...more

U.S. Department of Energy Makes Conditional Commitment to Provide HALEU for Hermes Demonstration Reactor



Kairos Power



U.S. DEPARTMENT of ENERGY



Research lab aims to field

tron microreactor, is partnering with research and Université Paris-Saclay to chemistry which aims to be the European nuclear molten salt reactors and non-

reactor to streamline the pre-license applications expected next year.

Regulation & Safety · Wednesday, 4 September 2024

Review for Thorizon One

nds has announced that the regulators are to collaborate of its Thorizon One molten salt



NATURA MSR-100 REACTOR FACILITY

Denmark's Copenhagen Atomics has signed a large-scale experimental collaboration agreement with Switzerland's Paul Scherrer Institute to conduct the first critical experiment on thorium molten salt reactors in Europe.

New Nuclear · Monday, 1 July 2024



Molten Salt Reactor
PROGRAM



U.S. DEPARTMENT of ENERGY
Office of Nuclear Energy

MSR Developers Status

Construction Permit Issued in December 2023 and Construction Started



Hermes 1*

Construction Permit issued in September 2024



Molten Salt Research Reactor

Construction Permit Issued in November 2024



Hermes 2*

Pre-Application Document Reviews Ongoing



IMSR (Integral Molten Salt Reactor)

Other Vendors that have had Early Engagement with NRC

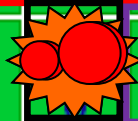


*TRISO fuel

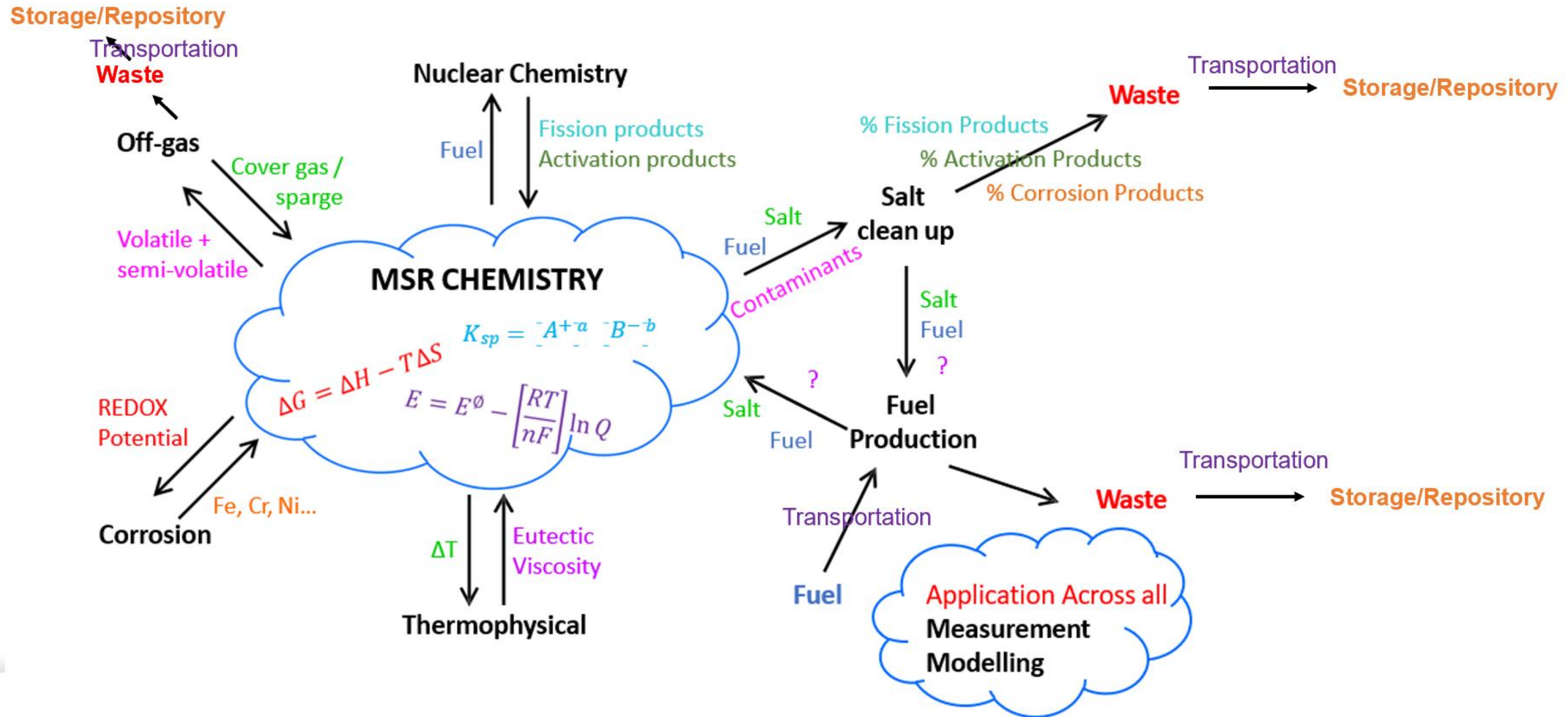
<https://www.nrc.gov/reactors/new-reactors/advanced/who-were-working-with/pre-application-activities/kairos.html>

Irradiated Nuclear Fuel: Chemical Diversity

1 H																	2 He	
3 Li	4 Be											5 B	6 C	7 N	8 O	9 F	10 Ne	
11 Na	12 Mg											13 Al	14 Si	15 P	16 S	17 Cl	18 Ar	
19 K	20 Ca	21 Sc	22 Ti	23 V	24 Cr	25 Mn	26 Fe	27 Co	28 Ni	29 Cu	30 Zn	31 Ga	32 Ge	33 As	34 Se	35 Br	36 Kr	
37 Rb	38 Sr		39 Y	40 Zr	41 Nb	42 Mo	43 Tc	44 Ru	45 Rh	46 Pd	47 Ag	48 Cd	49 In	50 Sn	51 Sb	52 Te	53 I	54 Xe
55 Cs	56 Ba	*	71 Lu	72 Hf	73 Ta	74 W	75 Re	76 Os	77 Ir	78 Pt	79 Au	80 Hg	81 Tl	82 Pb	83 Bi	84 Po	85 At	86 Rn
87 Fr	88 Ra	**	103 Lr	104 Rf	105 Db	106 Sg	107 Bh	108 Hs	109 Mt	110 Ds	111 Rg	112 Cn	113 Uut	114 Uuq	115 Uup	116 Uuh	117 Uus	118 Uuo
		*	57 La	58 Ce	59 Pr	60 Nd	61 Pm	62 Sm	63 Eu	64 Gd	65 Tb	66 Dy	67 Ho	68 Er	69 Tm	70 Yb		
		**	89 Ac	90 Th	91 Pa	92 U	93 Np	94 Pu	95 Am	96 Cm	97 Bk	98 Cf	99 Es	100 Fm	101 Md	102 No		




Chemistry is Everywhere in an MSR



Challenges

Chemistry Challenges

- Gaps in thermophysical and thermochemical properties of molten salts, Actinide fluoride and chloride bearing mixtures. No irradiation studies on chloride fuel salt .

Monitoring and Chemistry Control of Molten Salts, with potential Online Purification

- Fuel Salt Processing: The online processing of liquid fuel salts to extract fission products and manage the chemistry of the salt mixture is one of the most promising aspects of MSRs. However, this area needs further investigation and demonstration to ensure reliability and efficiency.

Safety Demonstration and Regulation

- Novel Designs and Fuel Cycles: MSRs employ innovative designs and fuel cycles, necessitating thorough safety demonstrations and regulatory approval. Developing safety cases and regulatory frameworks for MSRs is a key condition for its success.

Material Challenges

- MSRs operate at high temperatures and with molten salts for which corrosion can be an issue. Finding materials that can withstand this harsh chemical environment, resist corrosion, endure high temperatures, and tolerate neutron fluxes over long periods remains a significant challenge.
- Addressing these materials challenges requires extensive research and development. This includes testing and qualifying new alloys, composites, and coatings that can meet the demanding conditions of MSR operation.

Scale-up and Commercialization

- Scaling Up Technology: Transitioning from experimental or small-scale models to full-scale commercial reactors involves significant challenges in scaling up the technology and demonstrating its economic viability at a commercial level.



Molten Salt Reactor
P R O G R A M



U.S. DEPARTMENT
of **ENERGY**
Office of Nuclear Energy

VISION - The DOE-NE MSR campaign serves as the hub for efficiently and effectively addressing, in partnership with other stakeholders, the technology challenges for MSRs to enter the commercial market.



SALT CHEMISTRY

Thermophysical and Thermochemical Properties of Molten Salts – Experimental and Computational



TECHNOLOGY DEVELOPMENT

Off-Gas Management
Radionuclide Release Monitoring, Sensors & Instrumentation
LSTL & FASTR



NATIONAL AND INTERNATIONAL MSR SAFETY

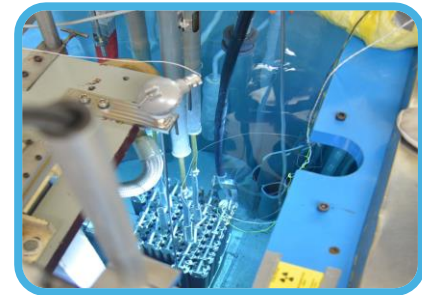
Guidance on reasonable approaches to demonstrating safe and economic commercial prototype molten salt reactors.

Mission: Develop the technological foundations to enable MSRs for safe and economical operations while maintaining a high level of proliferation resistance.

- 1) MSRs can provide a substantial portion of the energy needed for the U.S. to have energy security, energy independence.
- 2) There is a need for an abundant energy for the foreseeable future for data centers, desalination, process heat, hydrogen production...

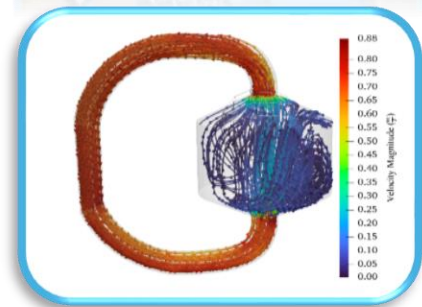
SALT IRRADIATION

Fuel salt irradiation and post irradiation examination capabilities
NRAD, ATR



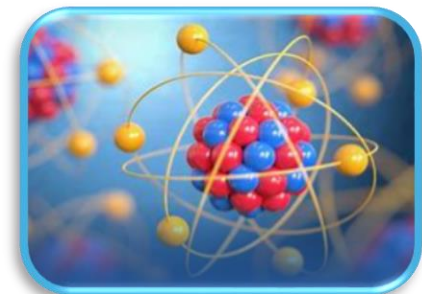
MODELING & SIMULATION

NEAMS Tools & MELCOR utilized for Species Tracking
Accident Scenario
Mechanistic Source Term...



MSR RADIOISOTOPES

Developing new Technologies to separate Radioisotopes of Interest to the MSR Community.



Molten Salt Reactor
P R O G R A M



U.S. DEPARTMENT
of ENERGY

Office of Nuclear Energy

Goals and Charter



Identify needs short- and long-term

Gather stakeholder input

Identify gaps being addressed , and assess remaining gaps

Identify action and resources needed



Share information

on R&D findings and share among stakeholder

Campaign review; MSR website, ORNL Annual MSR workshop



Inform and Complement

Relevant National and International Events (IAEA, NEA/OECD, GIF, Conferences, Workshops, Bootcamps...)



Input to roadmap

Develop a MSR roadmap which aligns with EPRI/NEI advanced reactor roadmap as well as ARSS and AMMT roadmaps

MSR Campaign Has Broad Objectives

SALT CHEMISTRY

Continue to develop capabilities for thermophysical properties of molten salts – Adding Vapor pressure and surface tension.
Continue development of an extensive thermal properties database (MSTDB-TP and MSTDB-TC). Workshop planned in June 2025
Data are used in multi-physics numerical codes for the simulations of the fuel salt heat and flow distribution in the reactor cavity.

RADIONUCLIDE RELEASE

Develop and demonstrate Online Monitoring technology to meet deployment process control needs . Transition Lab Scale to FASTR
Complete construction and dry shakedown of MSTTE (Molten Salt Tritium Transport Experiment) – Start - D₂/FLiNaK campaign
Complete design of engineering-scale spill test facility at ANL
Creation of a pumped actinide loop and supporting tools to enable the development and testing of molten salt reactor technologies. (loop to be installed at ANL to enable rapid installation of new components and changeover of salts)

IRRADIATION

FY24 a sodium chloride – uranium trichloride (NaCl-UCl₃) salt containing enriched uranium, was irradiated in the Neutron Radiography(NRAD) Reactor at INL.
Continue post irradiation examination (PIE) 1- neutron radiograph images, 2- precision gamma scanning (PGS), 3-. Emanation of material of construction for corrosion and salt interaction, 4- Analysis of the irradiated fuel for its chemical and isotopic composition.
Lessons learned during the first irradiation to be documented and used to guide the next design of the irradiation capsule.
Preliminary design of an improved irradiation capsule for ATR irradiation planned in FY 2027

MODELING

Deliver an initial engineering framework for species transport in MSR using NEAMS tools that includes all the main physics required for this purpose: neutronics modeling with thermal-hydraulics coupling and depletion, liquid-gas phase transfer, and species deposition and leaching in structures due to molecular and galvanic mechanisms. Start collaborative work with CEA for MSR safety.
MELCOR code analyses to evaluate/inform MSR campaign mechanistic source term for critical safety functions and generalized radionuclide species transport.

RADIO-ISOTOPES

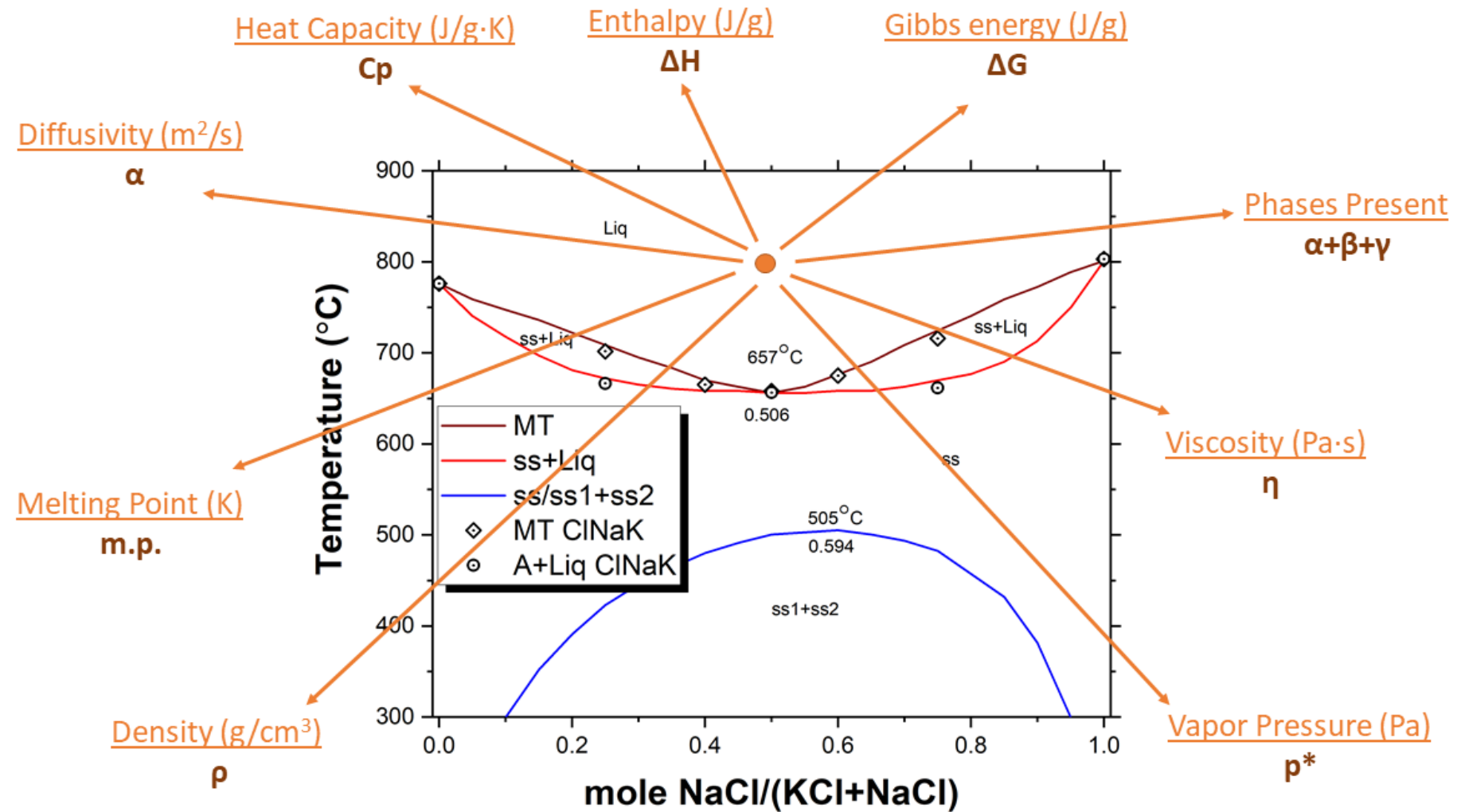
Chlorine Isotope Separations using Thermal Diffusion-
Finalizing the fabrication of the system, startup testing, production of highly enriched Cl-37 gas for use by other facilities and industrial partners.

SAFETY

Leadership on MSR safety and international cooperation in GIF MSR pSSC, GIF Safety/Safeguard activities, IAEA MSR technology, and CEA-DOE action plan on MSR Safety and development of a modern, non-proprietary evaluation model

Molten Salt Thermal Properties Determination – Experimental and Computational

- Density
- Heat Capacity
- Enthalpy of Fusion
- Viscosity
- Melting Point
- Vapor Pressure
- Thermal Conductivity
- Emissivity
- Surface Tension

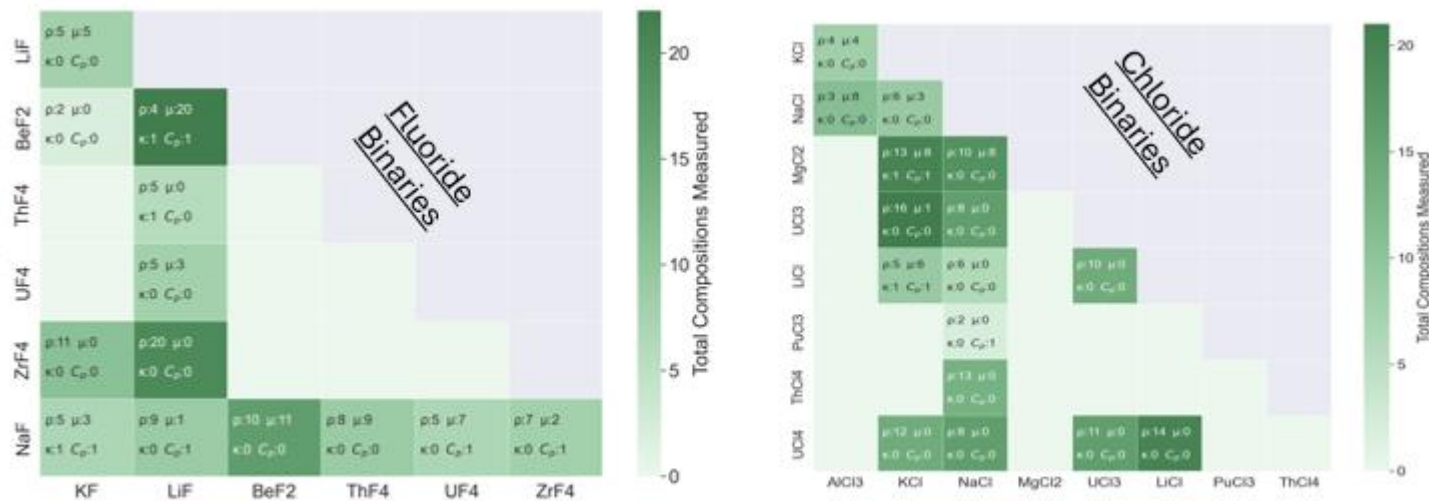


Molten Salt Thermal properties DataBase (MSTDB)

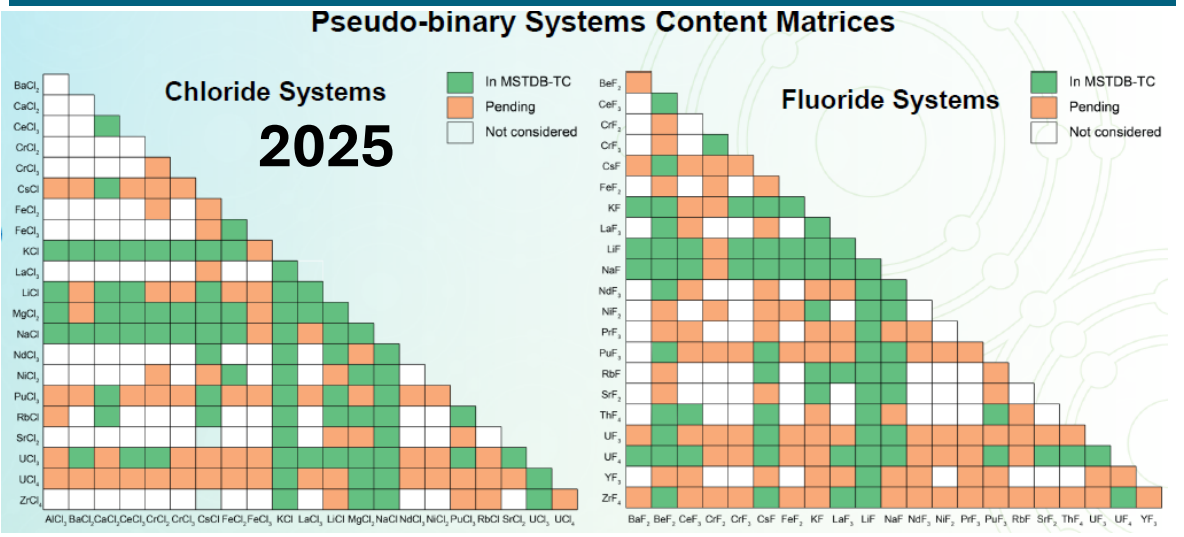
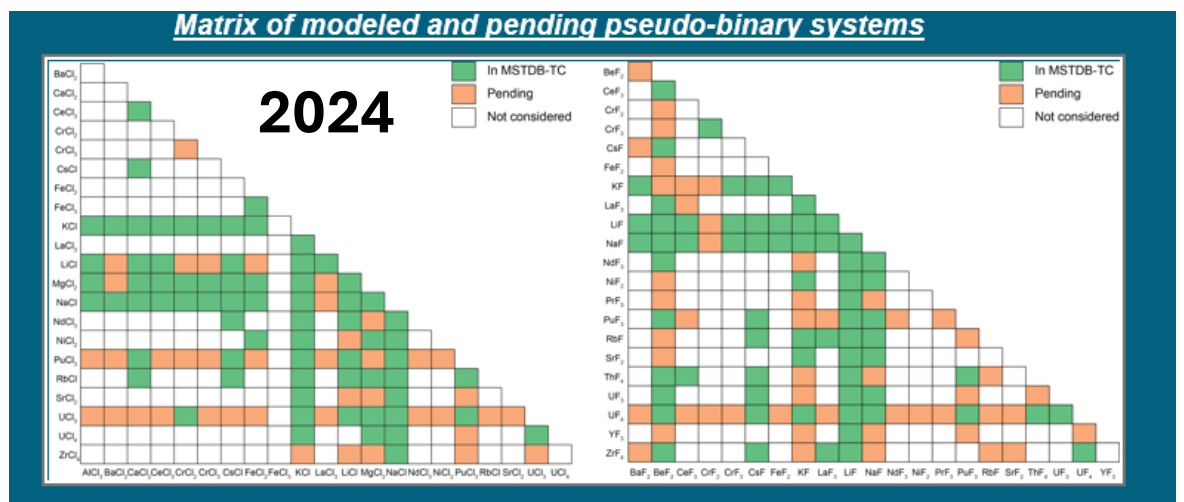
MSTDB-TP is managed by ORNL

MSTDB-TC is managed by U. of South Carolina

- MSTDB-TP contains empirical relations for the following properties:
 - Melting and boiling points; Density; Viscosity; Heat Capacity; Thermal Conductivity
- As per the current version release (3.1) There are **820 entries (FEB 2025)** including:
 - 33 pure compounds
 - 380 pseudo-binaries
 - 395 pseudo-ternaries
 - 12 pseudo-quaternaries
- Each property entry in the database includes a margin of experimental error
 - This list is constantly expanding. The data is based on the outputs of 180+ independent experimental studies in literature.



ORNL/TM-2024/3575



Recent release of the Molten Salt Thermal Properties Database-Thermochemical (MSTDB-TC) Ver. 4.0.

MSTDB Website

- New website to capture both MSTDB-TP and TC
 - Contains info about both arms
- Access links and instructions
- Recent News
- List of Publications and Workshop documents
- Contacts

Available @
<https://mstdb.ornl.gov>



[Home](#)
[About](#)
[Thermophysical \(TP\) ▾](#)
[Thermochemical \(TC\) ▾](#)
[Publications](#)
[News](#)
[Team](#)

Molten Salt Thermal Properties Database -- MSTDB

The Molten Salt Thermal Properties Database–Thermochemical (MSTDB-TC) and Molten Salt Thermal Properties Database–Thermophysical (MSTDB-TP) databases are available via the ORNL/ITSD Gitlab Server. MSTDB-TC contains Gibbs energy models and values for molten salt components and related systems of interest with respect to molten salt reactor technology. MSTDB-TP consists of tabulated thermophysical properties and relations for computing properties as a function of temperature or composition.

Pseudo-binary System

- FLUORIDES
- CHLORIDES
- IODIDES
- RECIPROCAL SALTS

Pseudo-ternary System

- FLUORIDES
- CHLORIDES
- IODIDES
- RECIPROCAL SALTS
- TERNARY RECIPROCAL SALT SYSTEMS

Higher-Order Systems

- FLUORIDES
- CHLORIDES
- RECIPROCAL TETRAHEDRA

ONE COMPREHENSIVE THERMAL PROPERTIES

Publications	
MSTDB Reference Documents/Publications	
THERMODYNAMIC MEASUREMENTS AND ASSESSMENTS FOR THE LiCl-NaCl-KCl-UCl ₃ SYSTEMS	▼
THERMODYNAMIC ASSESSMENTS OR REASSESSMENTS OF 30 PSEUDO-BINARY AND -TERNARY SALT SYSTEMS	▼
APPLICATION OF THE REDLICH-KISTER EXPANSION FOR ESTIMATING THE DENSITY OF MOLTEN FLUORIDE PSUEDO-TERNARY SALT SYSTEMS OF NUCLEAR INDUSTRY INTEREST	▼
DEVELOPMENT AND APPLICATION OF A THERMOCHEMICAL DATABASE (MSTDB-TC) FOR MODELING CORROSION IN MOLTEN SALT REACTORS	▼
THERMODYNAMIC ASSESSMENT OF LITHIUM HALIDE RECIPROCAL SALT SYSTEMS FOR ENERGY APPLICATIONS	▼
MODELING METALLIC HALIDE LOCAL STRUCTURES IN SALT MELTS USING A GENETIC ALGORITHM	▼
DEVELOPMENT OF THE MOLTEN SALT THERMAL PROPERTIES DATABASE - THERMOCHEMICAL (MSTDB-TC), EXAMPLE APPLICATIONS, AND LiCl-RbCl AND UF ₃ -UF ₄ SYSTEM ASSESSMENTS	▼
EMPIRICAL ESTIMATION OF DENSITIES IN NaCl-KCl-UCl ₃ AND NaCl-KCl-YCl ₃ MOLTEN SALTS USING REDLICH-KISTER EXPANSION	▼
CORRELATIONAL APPROACH TO PREDICT THE ENTHALPY OF MIXING FOR CHLORIDE MELT SYSTEMS	▼

MSTDB-TC

	Fluorides	Chlorides	Bromides	Iodides
Alkali metals	LiF, NaF, KF, RbF, CsF	LiCl		
Alkaline earth metal	BaF ₂ , CaF ₂ , SrF ₂ , BaF ₂	MgCl ₂ , CaCl ₂	BeF ₂ , MgF ₂	
Transition metals	NiF ₂ , CrF ₃	CrCl ₃ , CrCl ₂ , FeCl ₃ , FeCl ₂		
Other cations	VF ₅ , ZrF ₄	AlCl ₃		
Lanthanides	LaF ₃ , CeF ₃ , PrF ₃ , PbF ₂	CeCl ₃ , LaCl ₃		
Actinides	ThF ₄ , UF ₆ , UO ₂	UCl ₄ , UCl ₃ , PuCl ₃		
Pseudo-binary	70 systems	70 systems	30 systems	
Pseudo-ternary	30 systems	27 systems	15 systems	
Higher order	16 systems	2 systems	All 18 include iodides	

QUATERNARY AND HIGHER ORDER BIPRODUCT SALT SYSTEMS

NEW RELEASE: Molten Salt Thermal Properties Database-Thermochemical (MSTDB-TC) Ver. 3.0

The Molten Salt Thermal Properties Database-Thermochemical (MSTDB-TC) Ver. 3.0 (succeeding Ver. 2.0) is now available for general use. Access procedures remain the same as for earlier versions.

[Read more](#)

Molten Salt Thermal Properties Working Group Hosts MSTDB Workshop

The Molten Salt Thermal Properties Working Group hosted a virtual workshop on the MSTDB virtually on April 25, 2023.

[Read more](#)

Previous MSTDB workshop content publicly available

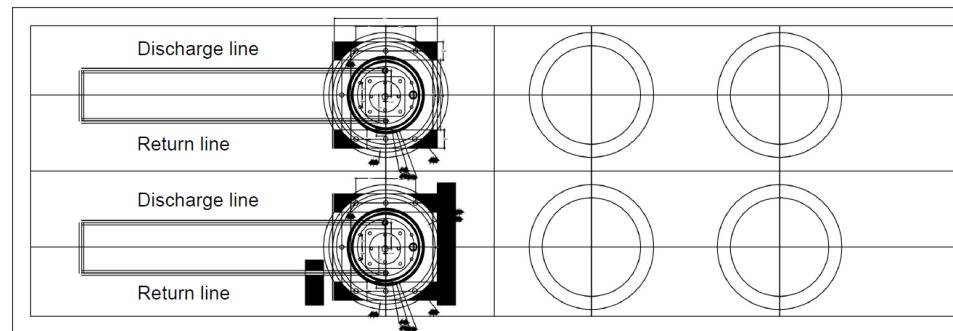
Example phase diagram of KCl-CrCl₂ from MSTDB-TC, presented by Dr. Ted Besmann in the 2021 Virtual Workshop for the Molten Salt Thermal Properties Working Group

[Read more](#)

Salt Loops supporting MSR R&D

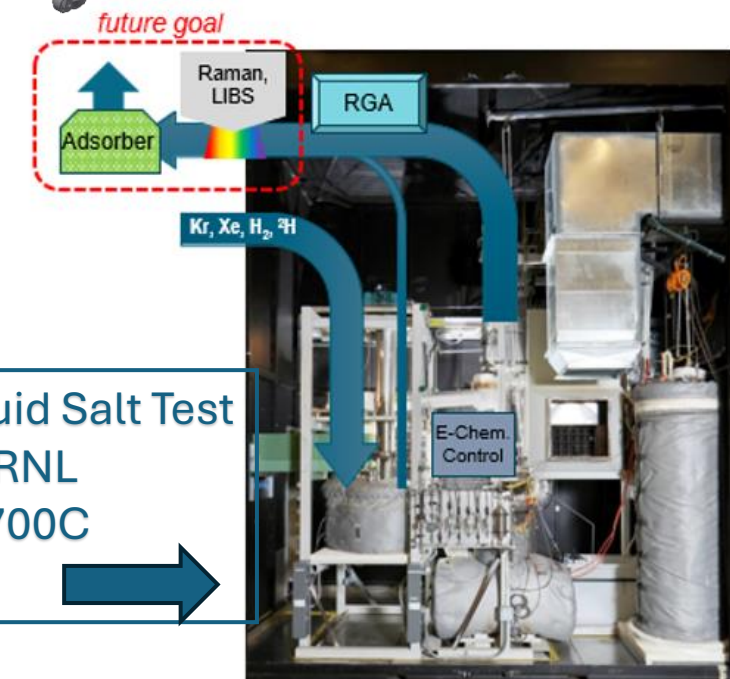
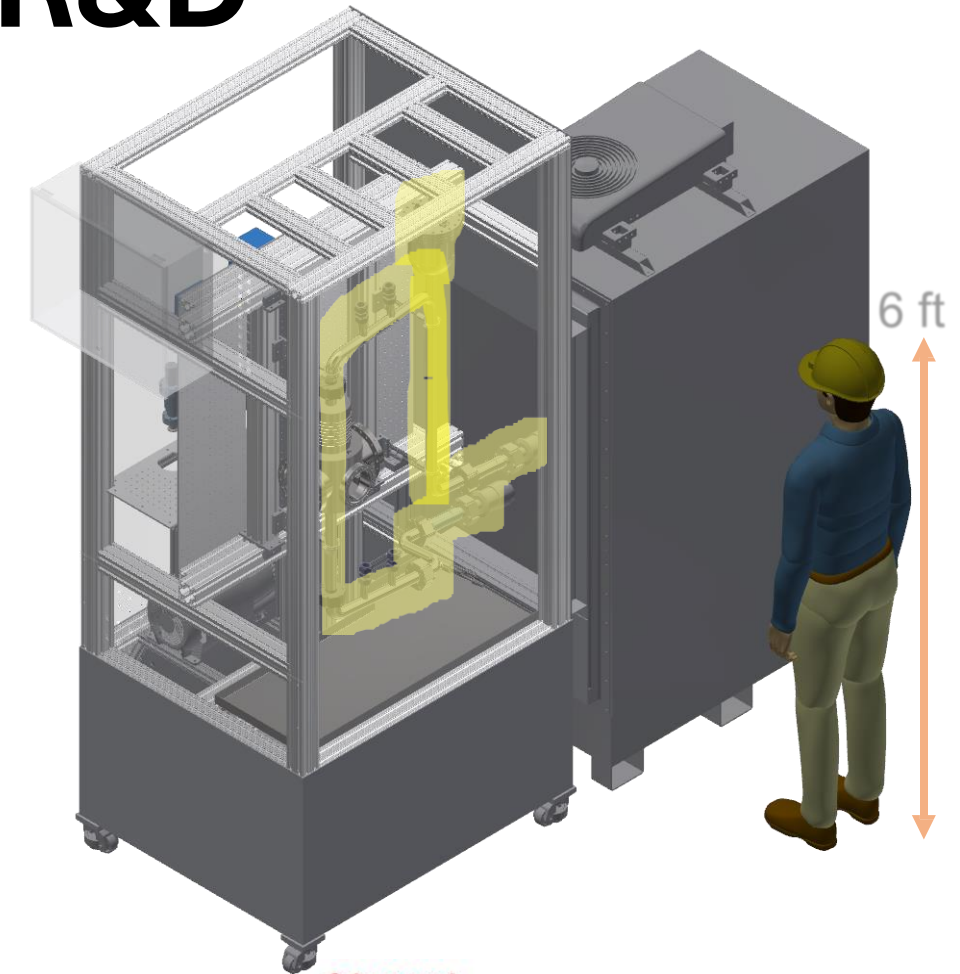
MSTTE – Molten Salt Tritium
Transport at INL
FY 2025 Deuterium- FLiNaK salt

Actinide Salt Loop at ANL



FASTR - Facility to Alleviate Salt
Technology Risks – ORNL
NaCl-KCl-MgCl₂
725C
154L

LSTL - Liquid Salt Test
Loop at ORNL
FLiNaK – 700C
80L



Technology Development and Demonstration

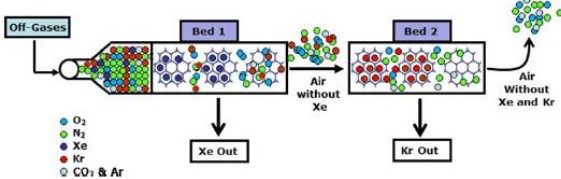
Multi-faceted approach to investigation of technologies for MSR off-gas systems

Component Testing

Large Scale Test Loop



Xe/Kr separation in MOF



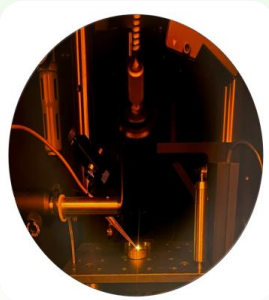
Radionuclide Identification/Speciation

Raman Spectroscopy



405 nm 532 nm 671nm

Laser Induced Breakdown Spectroscopy



Gas/Liquid Interface

Bubble Measurement



Source Term Modeling

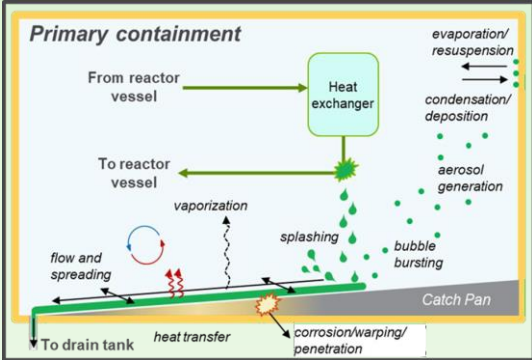
Bulk Gas	Gas Film	Liquid Film	Bulk Liquid
p_i pressure	p_i^*	c_i^*	c_i concentration

$$c_i^* = p_i H$$

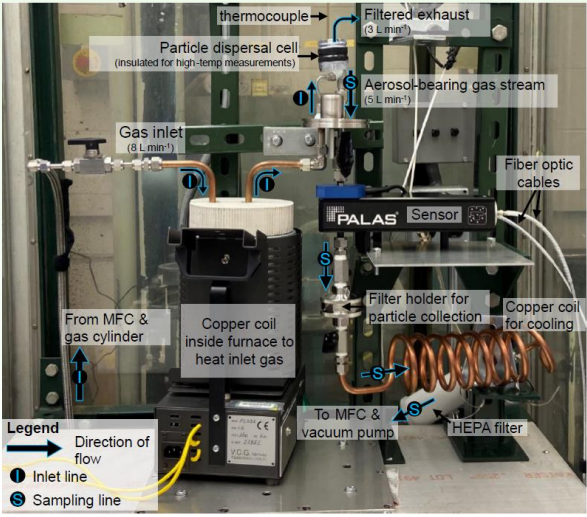
$$p_i^* = c_i / H$$

Spill Test

Engineering Scale Salt Accident Analysis Facility



Salt Aerosol Concentration and Size Measurements

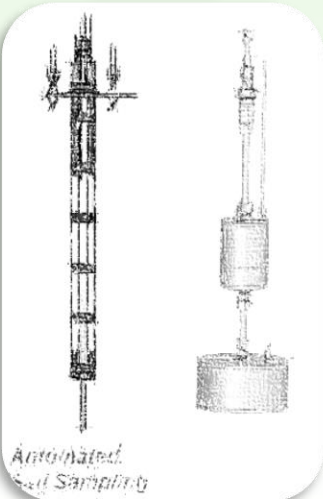


Sensors/Salt Chemistry

Salt Composition Redox Salt Level



Particulate Monitoring



Automated Salt Sampling

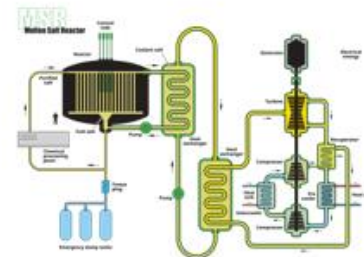
Modeling and Simulation

MELCOR: nuclear accident simulation code at SNL

Significant strides within the last few years to increase model accuracy for molten salt reactors (MSRs). It is the primary code developed for the

- Assessment and evaluation of safety for a broad range of reactor concepts, notably including MSRs.
- MSR model development has been focusing on improving chemistry and fission product transport mechanisms
- Working with MSR campaign to fill knowledge gaps

Molten Salt Tritium Transport Experiment



Integrated Experiments

Phenomenology Experiments



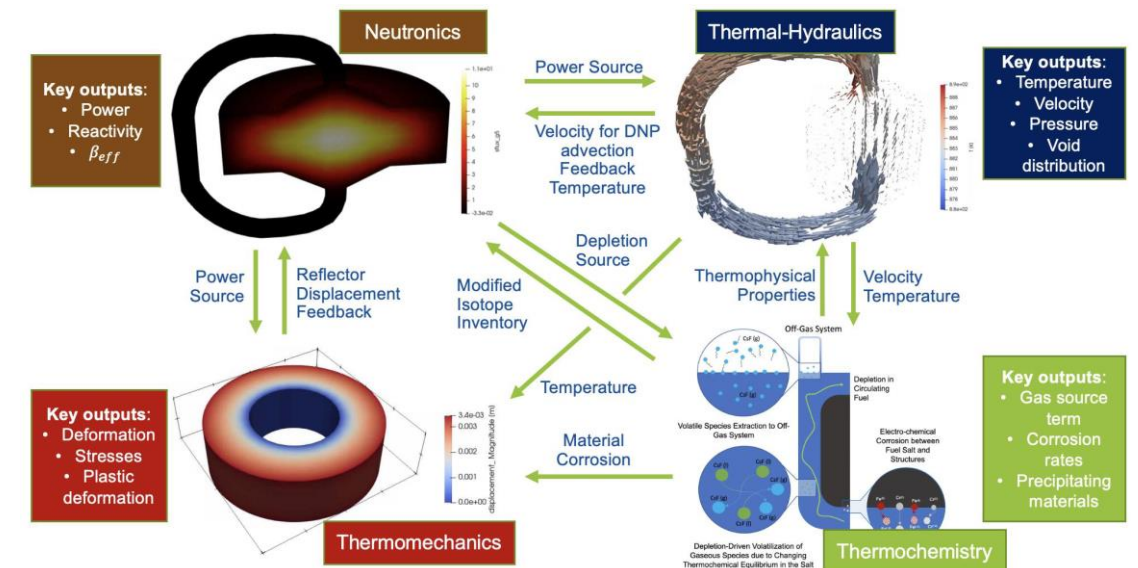
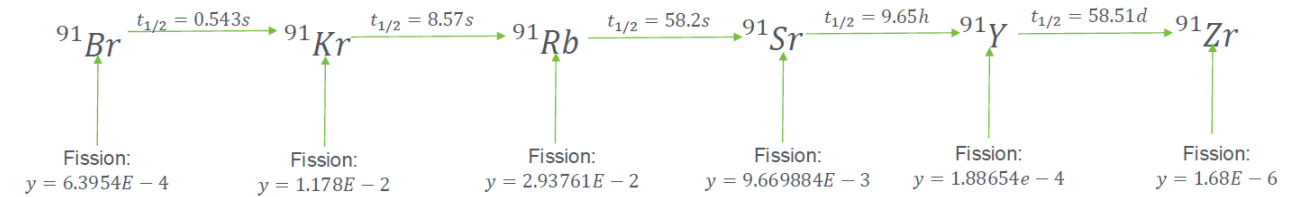
Liquid Salt Test Loop

MSR Simulation

NEAMS Tools at INL

Coupling between:

- Neutronics (multi-fidelity + spatial depletion),
- Thermal-Hydraulics (coarse-mesh CFD),
- Thermomechanics (thermo-elasto-plasticity with irradiation and thermal swelling and creep),
- Thermochemistry (speciation, mass accountancy, and corrosion)
- Species tracking in MSRs
 - Tracking the depletion chain of ^{91}Br



Molten-salt Research Temperature-controlled Irradiation (MRTI) Experiment Overview & Goals (INL)

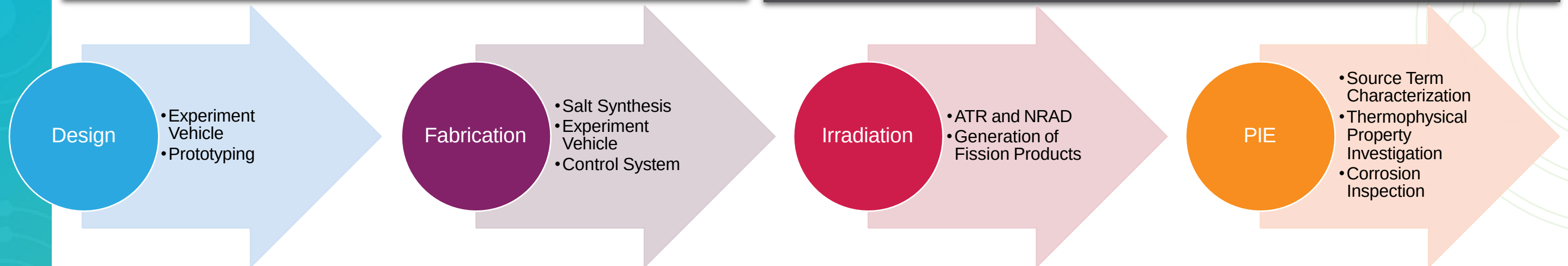
Establishment of a domestic neutron irradiation capability for fissile material-bearing salts at INL for Molten Salt Reactor (MSR) R&D.

Executing Research in Three Primary Areas

1. Radioactive Source Term Quantification
2. Thermophysical Property Evolution
3. Salt-facing Materials Corrosion

Mission Realization

Utilize the Neutron Radiography Reactor (NRAD) to irradiate molten fissile material-bearing chloride salt with salt-facing materials relevant to MSR development



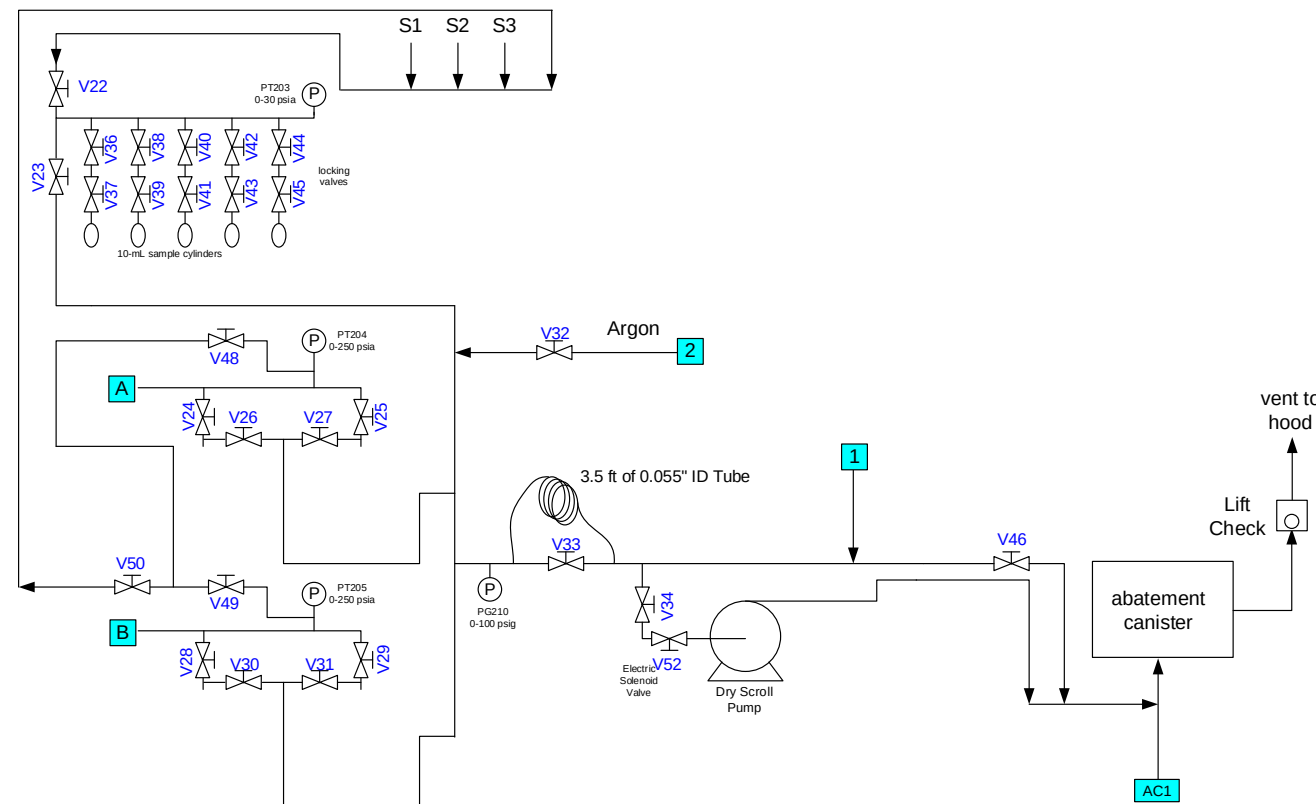
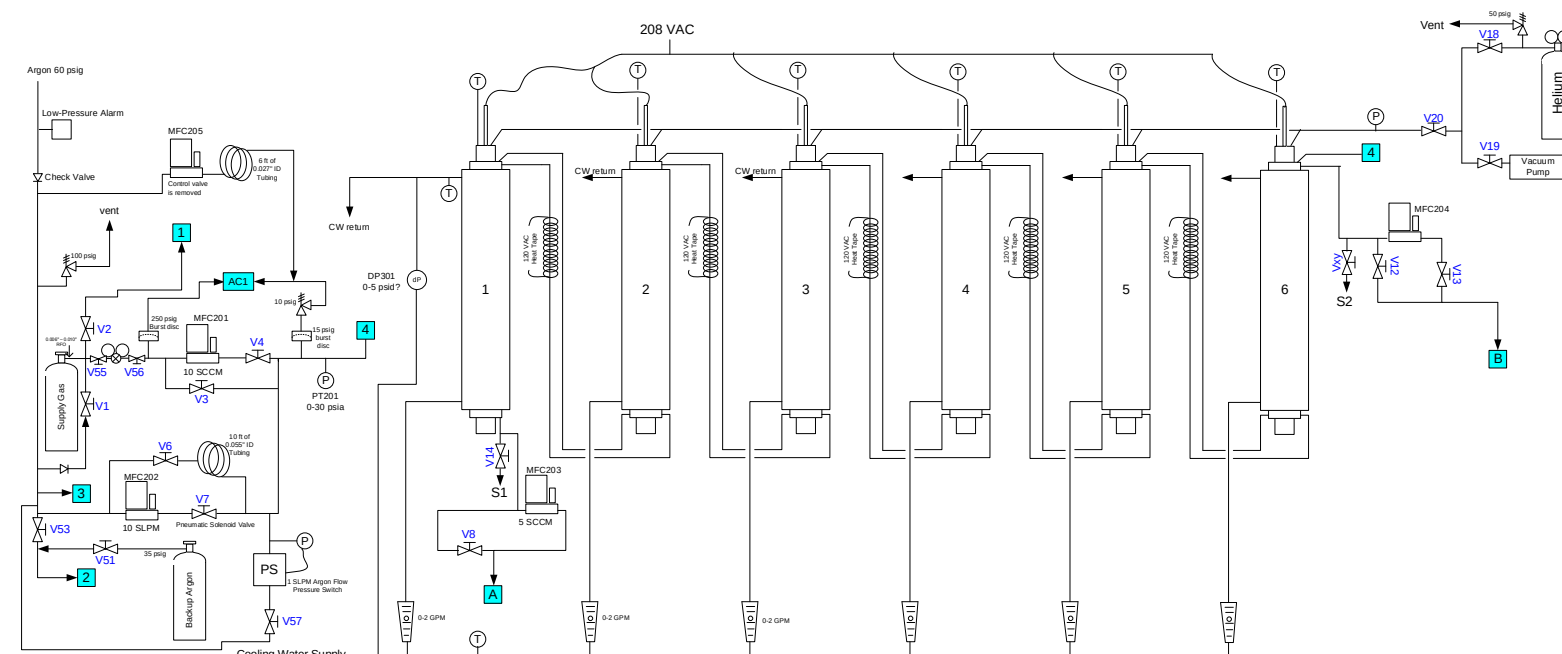
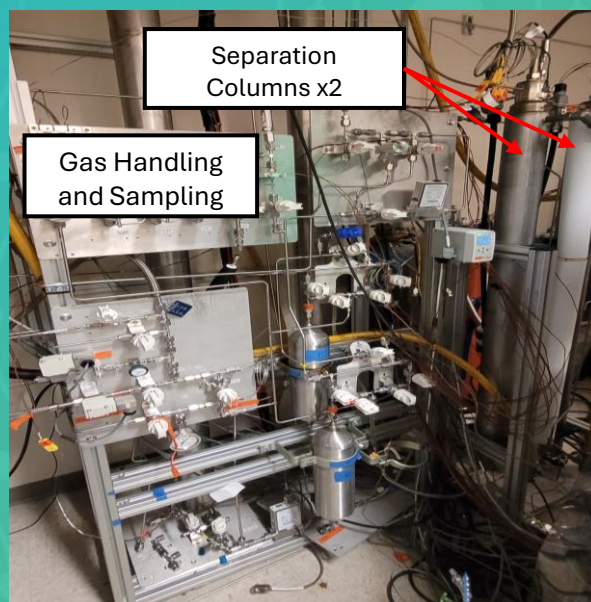
Increased interest and request for fuel salt irradiations and PIE from international vendors and several DOE programs. – Planning for next FYs at ATR and NRAD with MSR, AMMT, ARSS programs as well as NNSA – Off gas analysis to be performed at PNNL

Chlorine Isotope Separation System for Chloride MSR

- Thermal diffusion isotope separation system for enrichment of ^{37}Cl .
- Multi-physics model exists to optimize and inform facility designs at multiple scales
- Precise Cl isotope ICP-MS method with $\text{HCl}(\text{L})$ – no chemistry needed and $>1\%$ accuracy on $^{37}\text{Cl}/^{35}\text{Cl}$ ratio

WHY is Cl enrichment needed?

- ^{35}Cl (76% of natural chlorine) has large neutron capture cross section
- ^{36}Cl activation product is long-lived (301,000 years) and energetic (709 keV) beta emitter , highly soluble in water which will complicate waste form and final disposal
- Neutron irradiation of ^{35}Cl (n, p) ^{35}S - ^{35}S is more corrosive than Fe, Cr, Ni



HCl Columns Flow diagram

Courtesy B. McNamara, PNNL

International Engagement

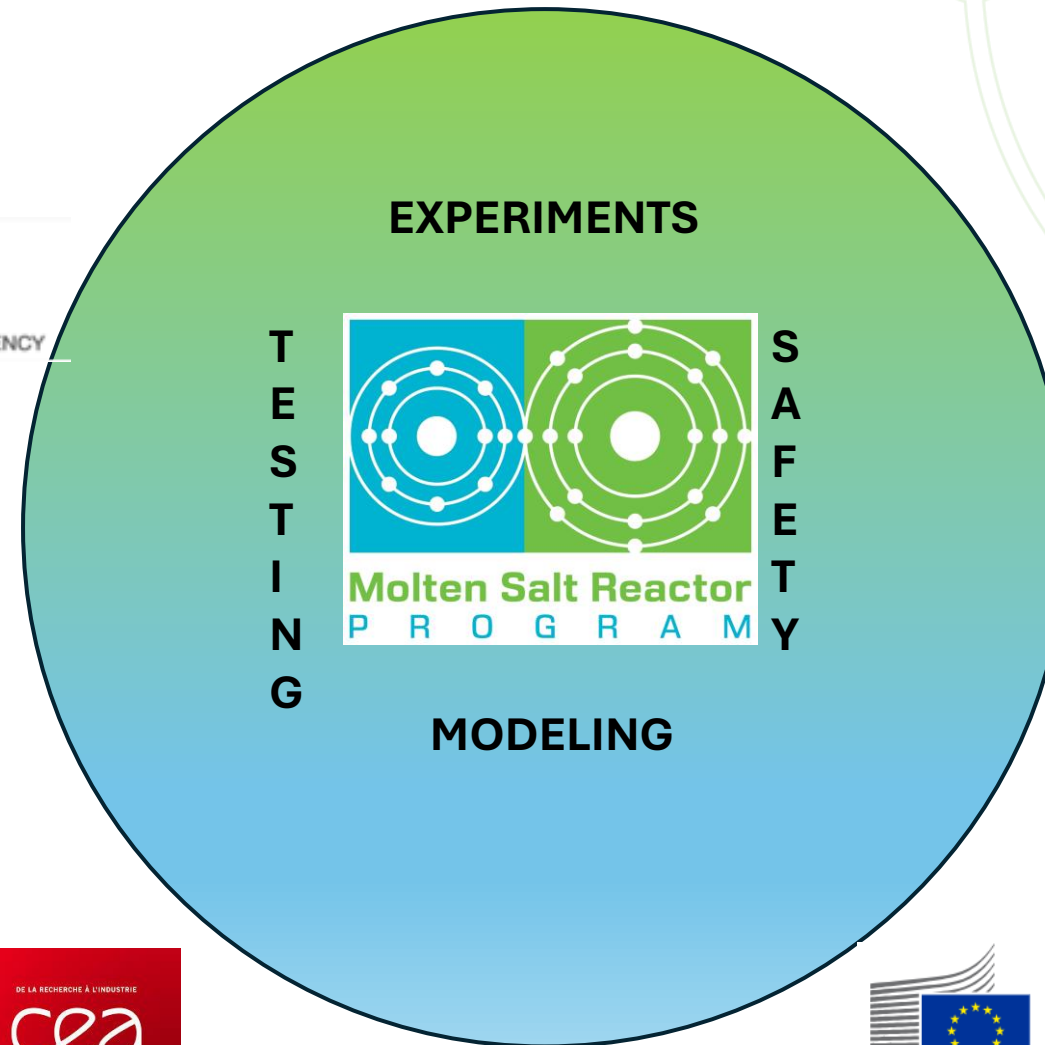


GIF MSR pSSC

13-15 MAY 2025 at Abilene Christian University, TX



CREAMM: CRITICAL EvAluation
of fuel Management during and
after MSR Operation - Kick off
meeting on 13 FEB 2025; in-
person meeting on 28-30 APR
2025 in Paris



Joint IAEA–GIF Workshop
on the Safety of Non-
Water Cooled Reactors
30 JUN-04 JUL 2025

IAEA Workshop - Molten
Salt Reactors Fuels:
Recent Development and
Future Challenges 21-25
JUL 2025

IAEA/NEA/EC-JRC joint
workshop on Molten Salt
Reactor (MSR) fuel cycle
taxonomy and related
terminology which will
take place between 3-7
NOV 2025 at the IAEA,
Vienna, Austria



DOE/CEA Action Plan
Safety of MSR



Molten Salt Reactor: Putting
Science into Standards March
2024

MSR Campaign Website:

<https://gain.inl.gov/doe-molten-salt-reactor-program/>

The breadth of the MSR design space presents a substantial challenge to the completeness and broad applicability of any technology development planning activity. Dozens of design concepts are currently in some state of development, nearly all have been introduced in the past decade, and it is not currently possible to reasonably evaluate which designs will eventually be successful. Nevertheless, MSRs have common characteristics and many technology development issues are broadly applicable to most MSRs

Contacts

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Office NE-52

MSR Annual Campaign Review

- [April 22-24, 2025](#)
- [April 16-18, 2024](#)
- [May 2-4, 2023](#)
- [April 26-27, 2022](#)
- [June 17, 2021](#)

MSR Course



Molten Salt Reactor Reports

CHEMISTRY

Announcements

Title	Link	Report Number	Year	# of Pages
Synthesis and Thermophysical Property Determination of NaCl-PuCl3 Salts	Link	INL/RPT-22-69181	2022	34
Engineering-Scale Batch Purification of Ternary MgCl2-KCl-NaCl Salt Using Thermal and Magnesium Contact Treatment	Link	ORNL/TM-2022/2554	2022	31
An Overview of the Molten Salt Thermal Properties Database□Thermophysical, Version 2.1.1 (MSTDB-TP v.2.1.1)	Link	ORNL/TM-2023/2955	2023	25
FY23 Progress Report on Viscosity and Thermal Conductivity Measurements of Molten Salts	Link	ORNL/TM-2023/3048	2023	31
Experimental Plan for Synthesis of an Americium- and Plutonium-Containing Salt	Link	INL/RPT-24-80052	2024	18

Thank you
Teresa Krynicki, INL

2025 Molten Salts Thermal Properties Working Group Virtual Workshop - *Effect of Oxygen, Hydrogen, and Moisture on Molten Salt Behavior*

10AM to 3PM EDT, 3-4 JUN, 2025 – Information and Link at MSTDB.ornl.gov

Save the date

Annual MSR Workshop, Knoxville, TN - 18-20 NOV 2025

**Next MSR Campaign Review in New Mexico, hosted by LANL
and SNL – April 2026**

GIF WEBINARS : <https://www.gen-4.org/resources/webinars>

Webinar #9 <https://www.gen-4.org/resources/webinars/education-and-training-series-9-molten-salt-reactors-msr>

Webinar #8 <https://www.gen-4.org/resources/webinars/education-and-training-series-8-fluoride-salt-cooled-high-temperature-reactors>

Webinar #35 <https://www.gen-4.org/resources/webinars/education-and-training-series-35-czech-experimental-program-msr-technology>

Webinar #42 <https://www.gen-4.org/resources/webinars/education-and-training-series-42-comparison-16-reactors-neutronic-performance>

Webinar #44 <https://www.gen-4.org/resources/webinars/education-and-training-series-44-molten-salt-reactor-safety-evaluation-us>

Webinar #73 <https://www.gen-4.org/resources/webinars/education-and-training-series-73-molten-salt-reactors-taxonomy-and-fuel-cycle>

Webinar #88 <https://www.gen-4.org/resources/webinars/education-and-training-series-88-multiphysics-depletion-chemical-analyses-molten>

Webinar #97 <https://www.gen-4.org/resources/webinars/education-and-training-series-97-overview-and-update-msr-activities-within-gif>



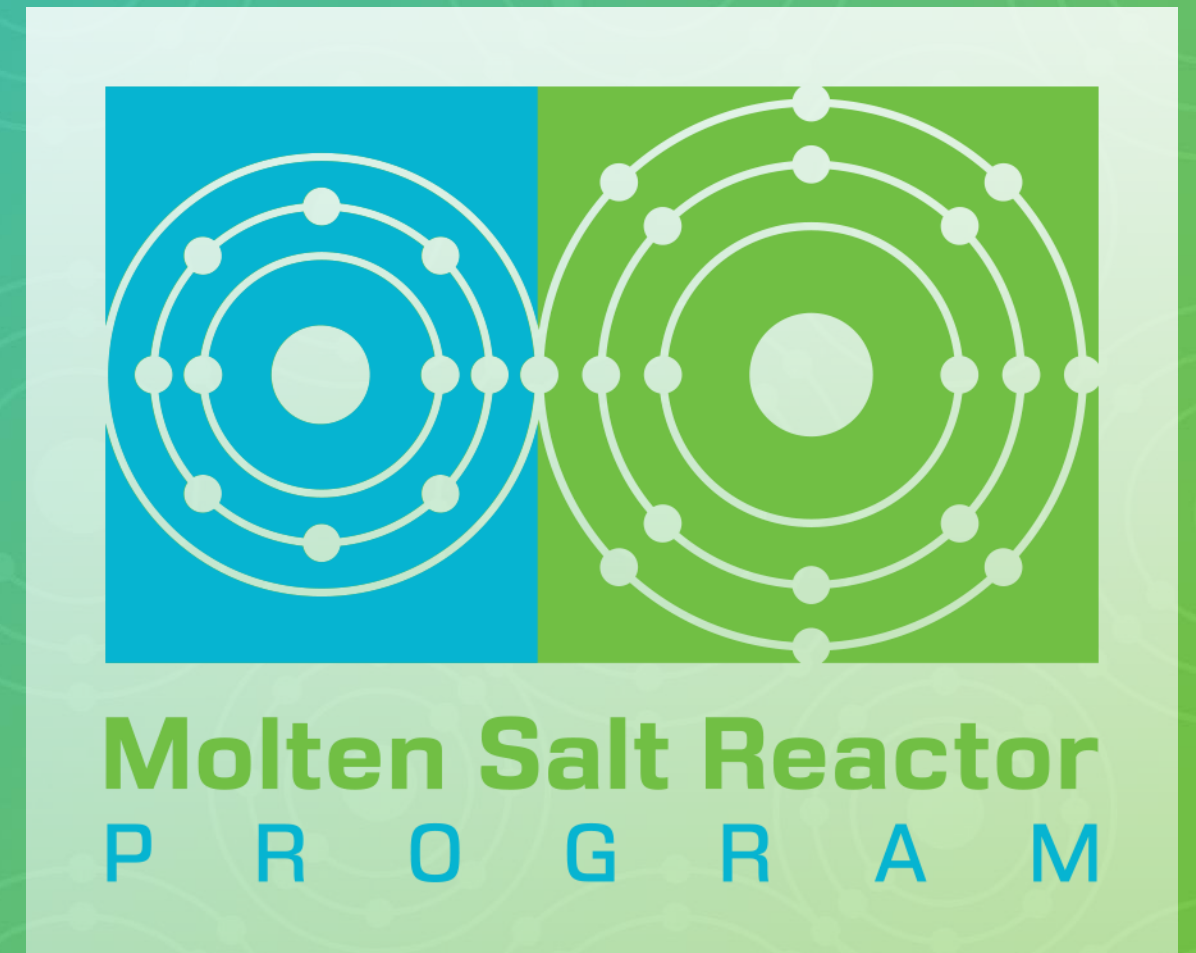
**Molten Salt Reactor
P R O G R A M**



**U.S. DEPARTMENT
of ENERGY**
Office of Nuclear Energy

Thank You

Patricia.Paviet@pnnl.gov



<https://gain.inl.gov/doe-molten-salt-reactor-program/>