



U.S. DEPARTMENT OF
ENERGY

Nuclear Energy

System Analysis Module (SAM)

Rui Hu

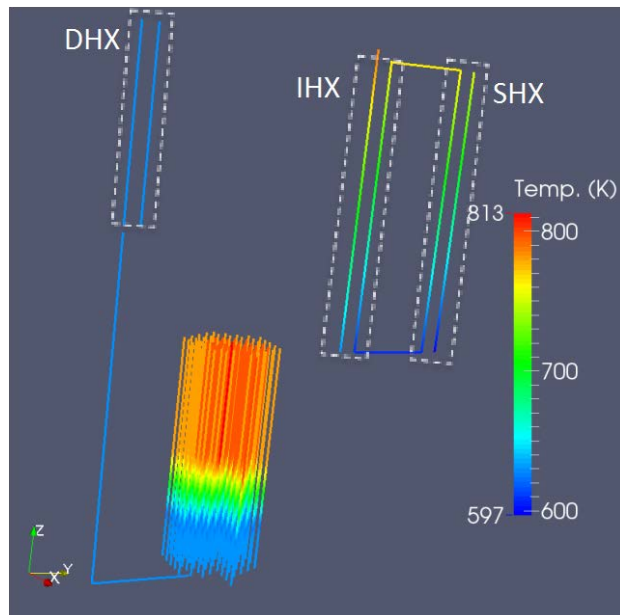
**Nuclear Engineering Division
Argonne National Laboratory**

**GAIN-EPRI Advanced Reactor M&S Workshop #2
EPRI Charlotte Campus
January 24-25, 2017**

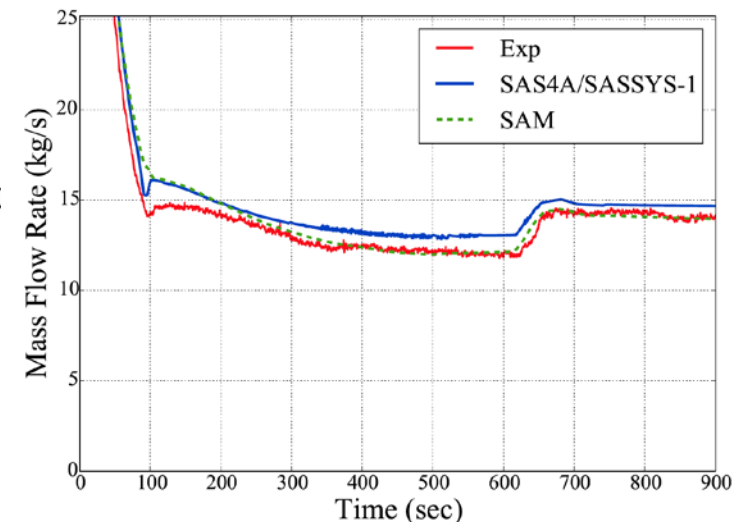
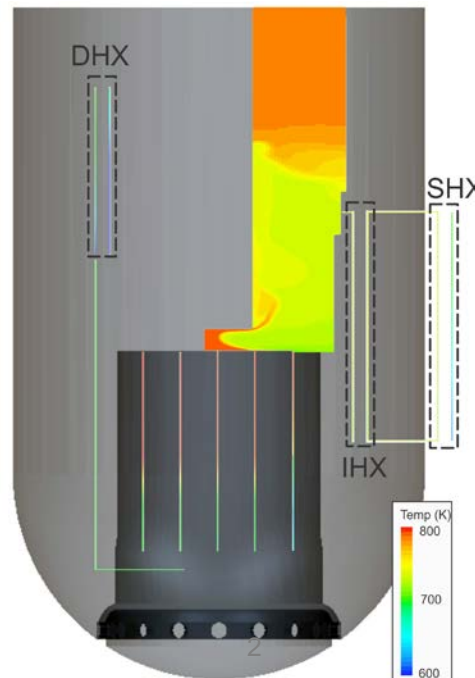


Overview

- A practical plant-level system analysis tool for advanced reactors (SFR, LFR, MSR/FHR);
- Advances in software environments and design, numerical methods, and physical models;
- Flexible multi-scale multi-physics integration with other high-fidelity tools.



Simulation of ABTR PLOF

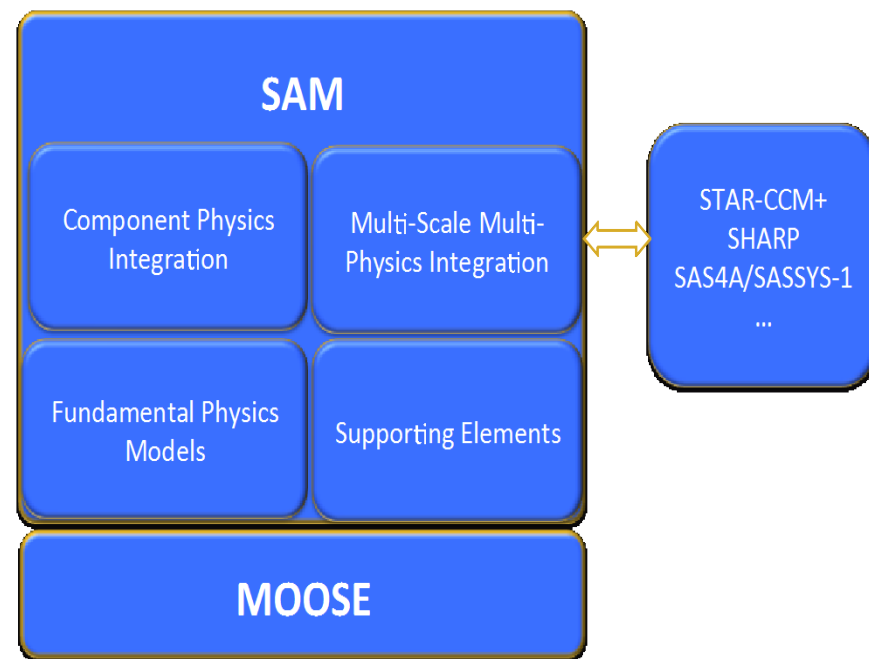


EBR-II SHRT-45R Benchmark



Development Approach

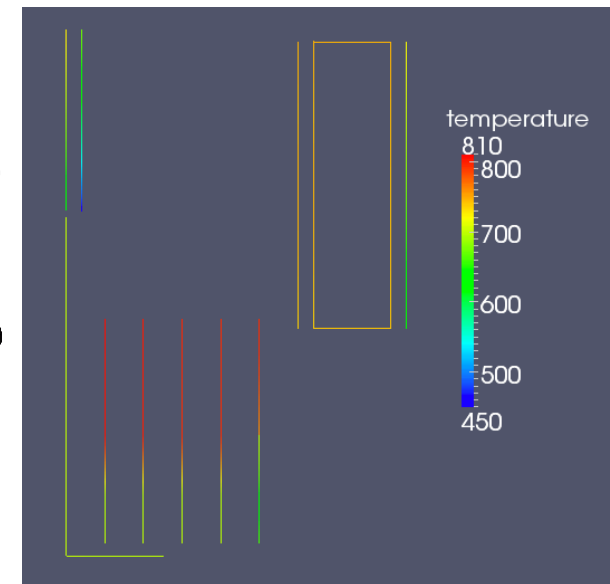
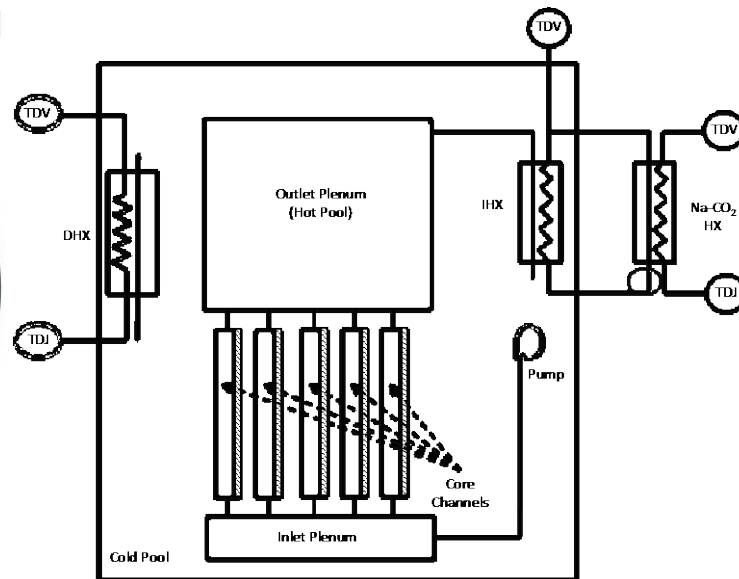
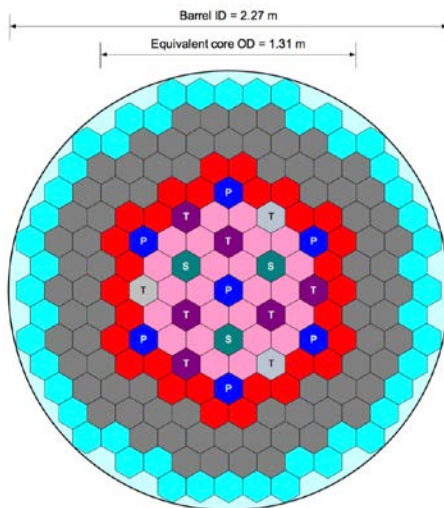
- Built-on MOOSE framework and other modern software libraries
 - Advanced solution methods
- Modern code development practice
 - Object-oriented C++ code
 - Modular, easy to develop and incorporate new capabilities
 - Designed for easy code reuse and code maintenance;
- Component based development
 - User friendliness
- Flexible for coupling with other M&S tools for multi-scale multi-physics modeling.





System Thermal-Hydraulics Modeling

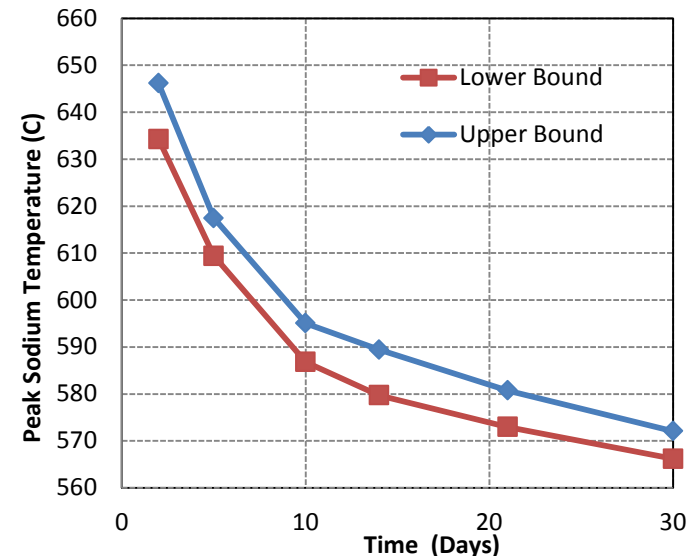
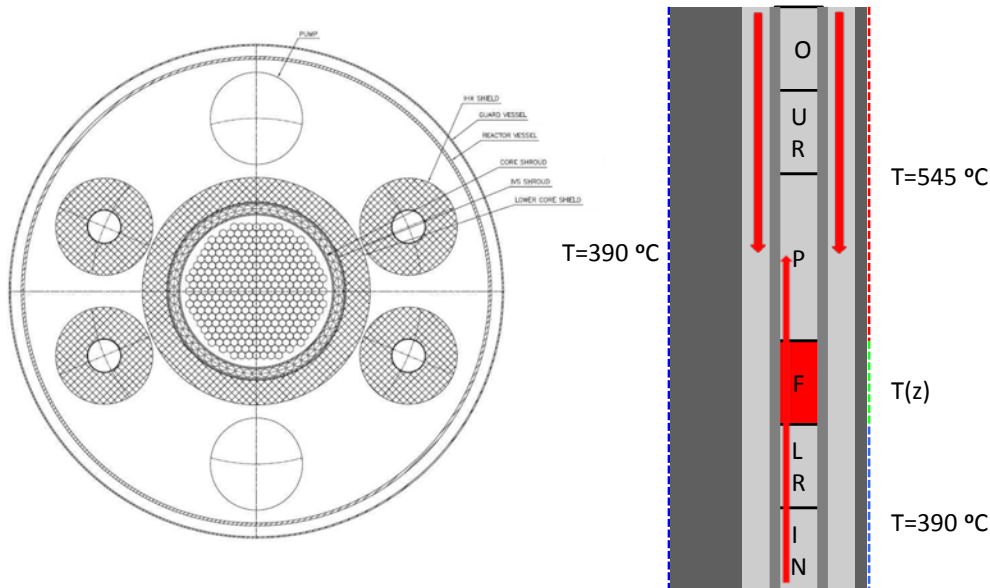
- Robust and high-order FEM model of single-phase fluid flow and heat transfer has been developed and verified;
- Component-based code development and system modeling;
- Flexible coupling capability between fluid and solid components enables a wide range of engineering applications;
- Closure Model Enhancements.





Application to an SFR IVS Cooling Analysis

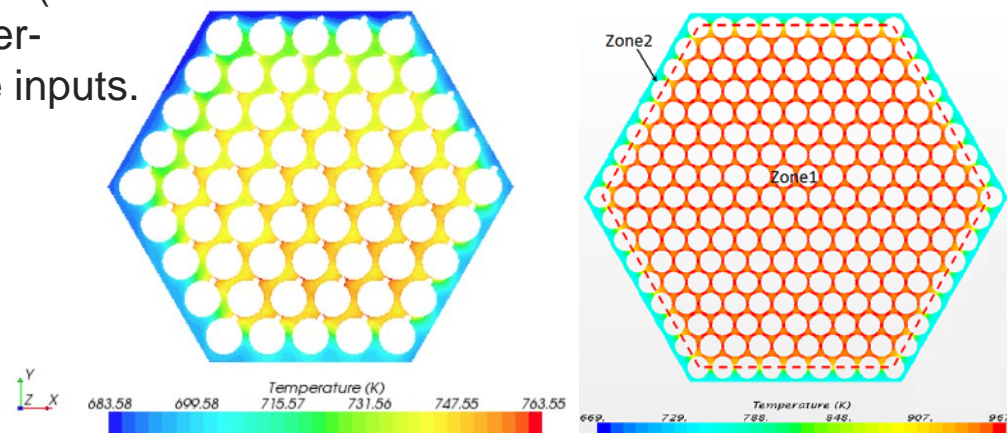
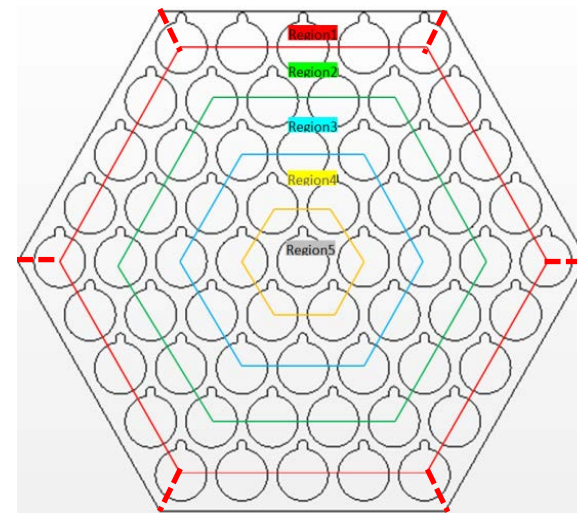
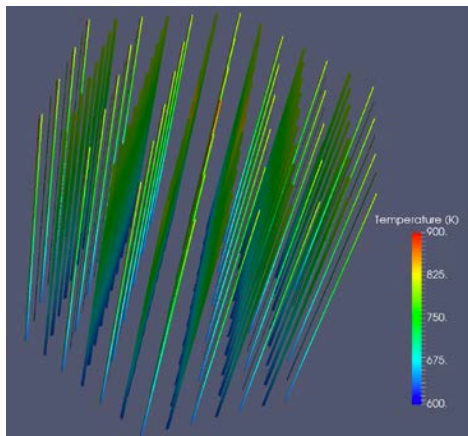
- In-Vessel Spent Fuel Storage design relies on natural convection and conduction to remove the decay heat in the spent fuel assemblies.
- SAM is uniquely suitable due to its flexibilities, accuracies and efficiency for modeling the conjugate heat transfer and natural circulation.
- Heat transfer to all major solid structures must be modeled, including the assembly duct wall, the core barrel, the IVS shroud, and the core shielding.
- Timely analysis assured the safety of the spent fuel storage, also avoided the potential design changes of the IVS.





SAM Core Modeling Options

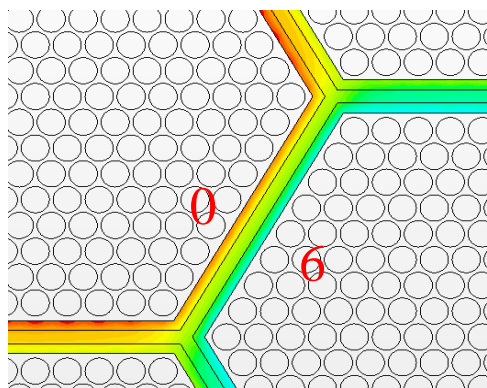
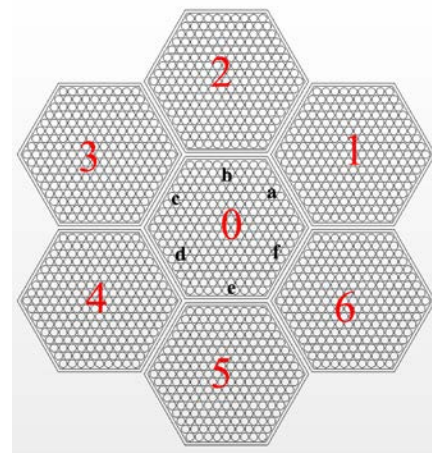
- Single-Channel Modeling
- Multi-Channel Modeling
 - Enabled by flexible fluid-solid CHT model in SAM;
- Flexible Full Core Modeling
 - HexLatticeCore component developed for SFR core modeling;
 - Automatically generate the core lattice (Multi-Channel or Single-Channel), and inter-assembly structures based on simple inputs.



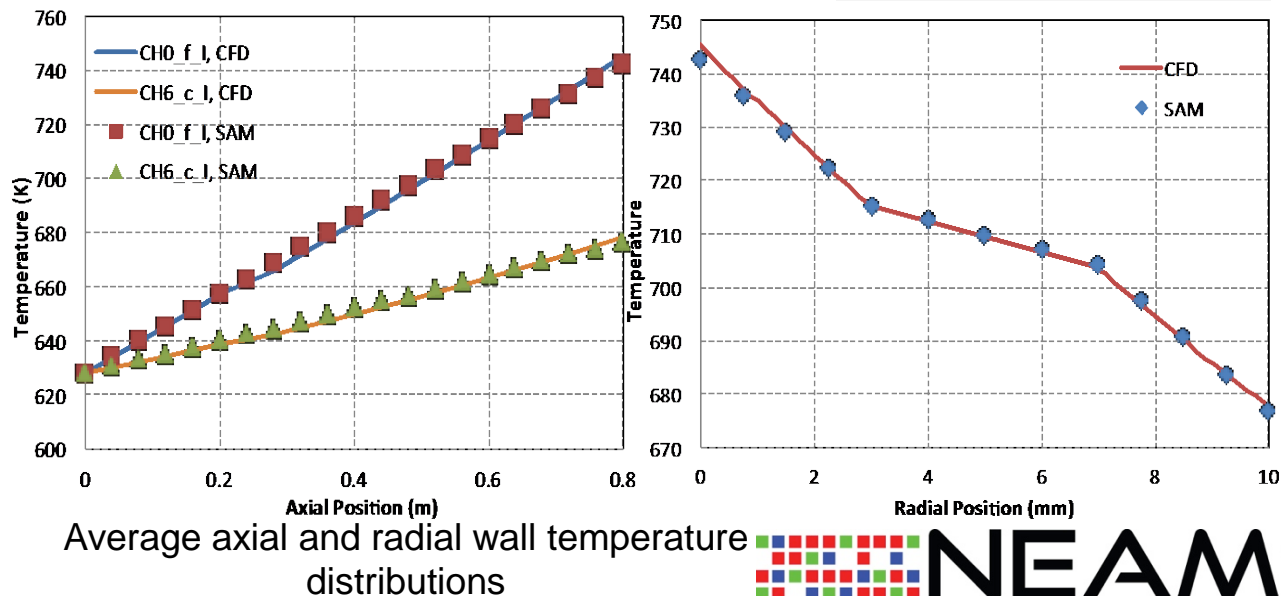


Virtual 3D Full-Core Conjugate Heat Transfer Modeling

- 7-Assembly demonstration problem with high power Assembly 0, and low power Assembly 6
 - The six sides of the hexagon duct wall of one assembly are modeled separately;
 - Multi-channel assembly model is used;
 - Very good agreement with CFD results, while the computational cost is reduced by 5-6 orders of magnitude.



CFD results of duct wall temperature distributions





Computationally Efficient Multi-Dimensional Flow Model

Objectives: Model the multi-dimensional flow and thermal stratification phenomena in large enclosures (outlet plena) for reactor safety analysis.

- Multi-dimensional conservation equations on a coarse mesh;
- No turbulence modeling;
- Developing closure models for inter-cell interaction and wall frictions;
- Additional diffusive term in the energy equation to account for the turbulence effects and the use of coarse mesh.

$$\frac{\partial \rho}{\partial t} + \nabla \cdot (\rho \vec{v}) = 0$$

$$\frac{\partial (\rho \vec{v})}{\partial t} + \nabla \cdot (\rho \vec{v} \vec{v}) = -\nabla p + \nabla \cdot \bar{\bar{\tau}} + \rho \vec{f}$$

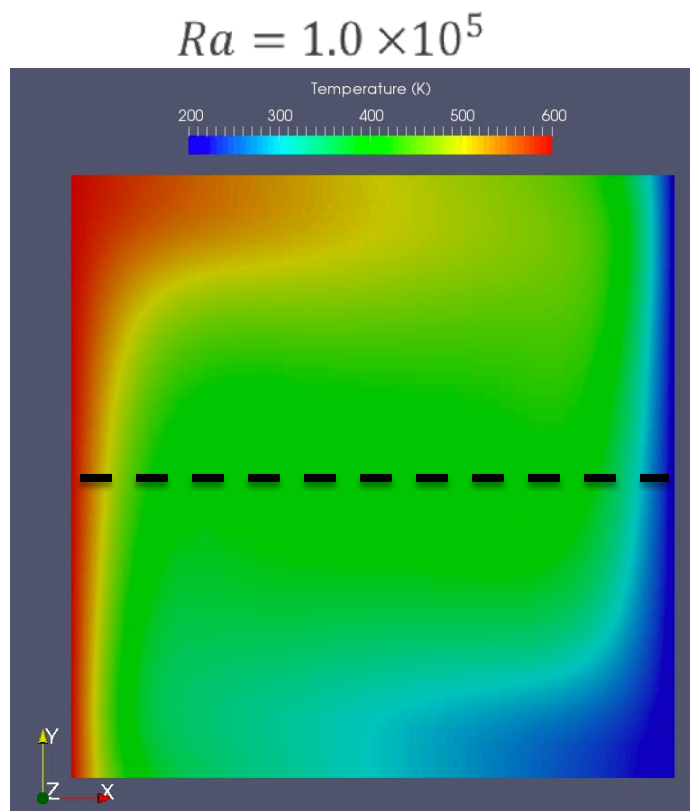
$$\frac{\partial (\rho h)}{\partial t} + \nabla \cdot (\rho h \vec{v}) = \nabla \cdot (k_{eff} \nabla T) + q_v'''$$

$$\rho = \rho(p, T)$$

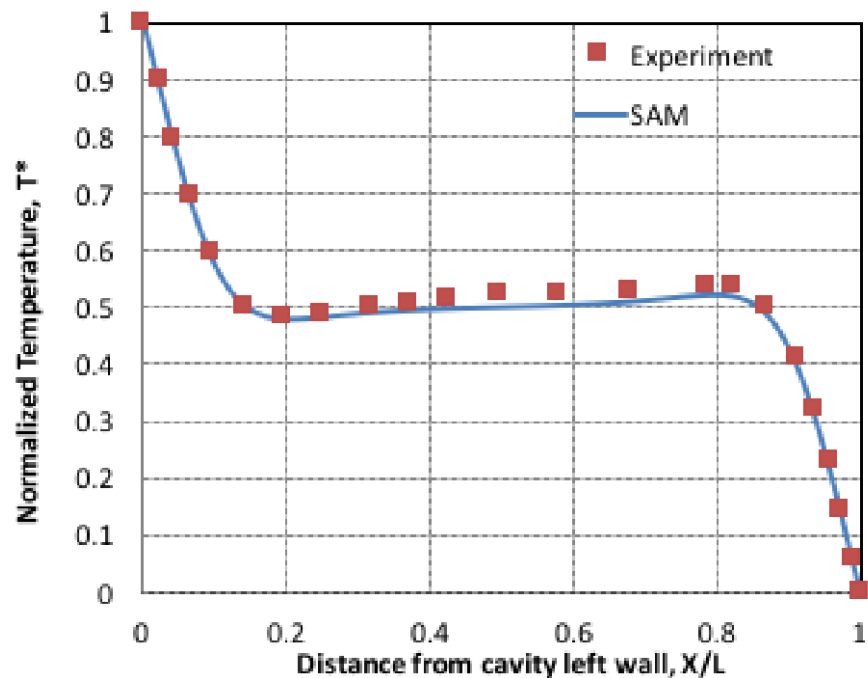


Preliminary Results from Verification Tests

- Test Problem: natural convection in a square cavity
 - Fixed temperatures on left and right walls; adiabatic top and bottom walls;



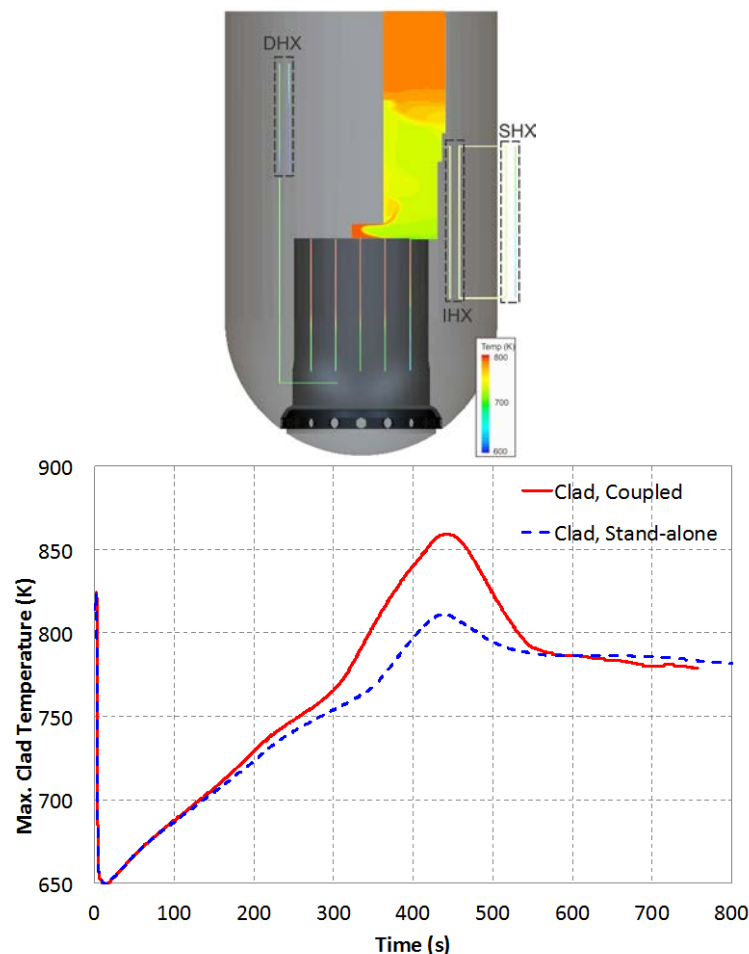
Normalized temperature distributions,
 $Ra = 1.89 \times 10^5$, $Y/L = 0.5$





Multi-Physics Multi-Scale Integration

- Flexible multi-scale multi-physics integration through coupling with other M&S tools
- Coupling with CFD codes
 - STAR-CCM+
 - Nek5000 (under development)
- Integration with SAS4A/SASSYS-1
 - Joint collaboration between DOE-NE's ART and NEAMS programs
- Planned Additional Coupling
 - PROTEUS
 - SHARP/SAM/Bison/Workbench

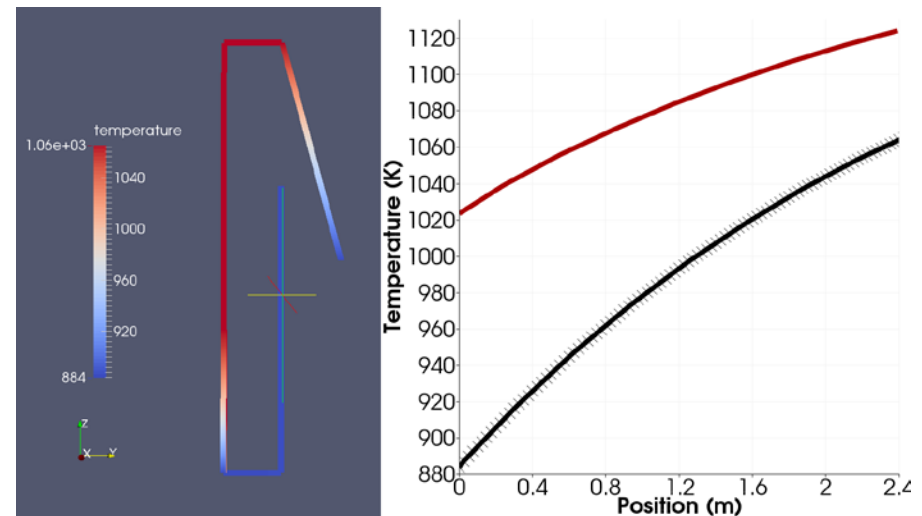




Enhancements for for MSR/FHR modeling

- Engagements with FHR IRPs under DOE-NE's NEUP program
 - FHR system T/H modeling
 - Code benchmarks
- NEUP project on salt freezing modeling
 - 1D freezing module in SAM
- Argonne LDRD project on MSR modeling
 - Radiation heat transfer
 - Liquid fuel transport in the primary loop
 - Additional closure models for salt fluid

SAM Model of PB-FHR DRACS



(Work performed by K. Ahmed (UWM) under the support of NEUP fellowship)



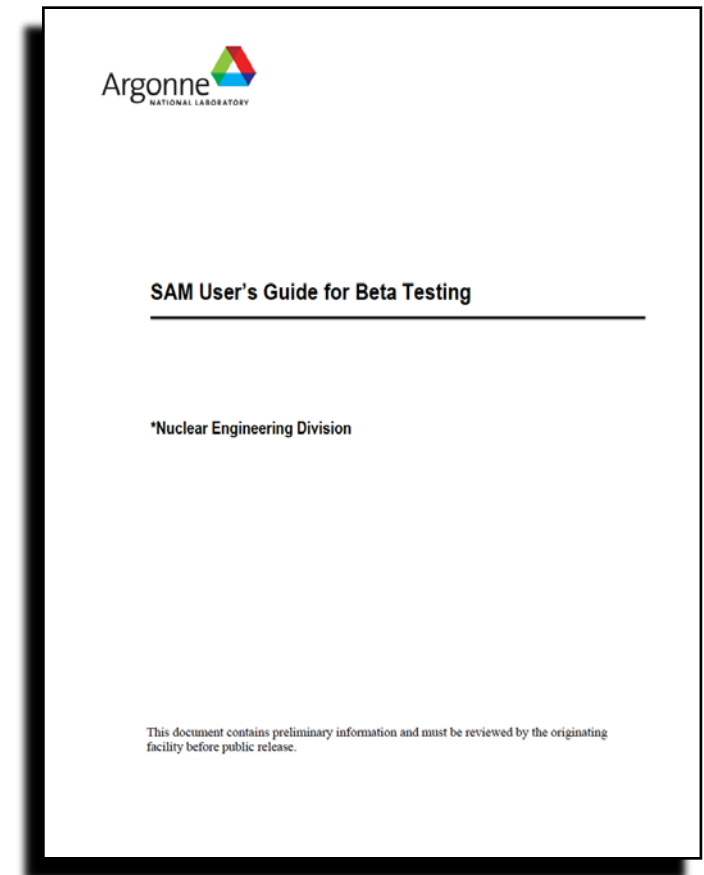
SQA and V&V

- Following QA and verification approaches used by MOOSE and MOOSE-based applications.
 - Continuous development with version control
 - Unit testing
 - Line coverage
- Over 150 test problems have been developed and included in the regression test suite
- Software Verification
 - Regression, Benchmark Tests, and Bug Reporting
 - Documentation
 - Verification with Analytical Benchmarks
 - Verification with Code-to-Code Comparison
- Software Validation
 - Unit, Component, Subsystem, System, Integral tests
 - Validation Matrix and Gaps
 - EBR-II, FFTF, CIET, etc.



SAM User Engagements

- Usage
 - Availability/Computational resource
 - Input syntax/GUI
- Limited distribution only
 - Non-Disclosure Agreement
 - Special agreement with universities;
 - Users: UWM, OSU/UM, ORNL/PSU, Oklo Inc. etc.
- Free open-source license planned for STH (pipe-network) simulation capabilities
- Training/Tutorial
- Internal use at Argonne
 - KAERI SPP on PGSFR design
 - DOE programs: ART, AFC





Ongoing and Future Work

- Reduced-order model for thermal-stratification
- Enhancements for MSR/FHR modeling
 - NEUP project on salt freezing modeling
 - LDRD project on MSR modeling
- Continued capability enhancements and V&V
 - Additional Component models
 - User experience (usability)
 - Benchmark with FFTF and other IET tests
- Multi-scale multi-physics coupling with other M&S tools
 - SAM/SAS
 - SAM-Nek5000, SAM-PROTEUS
 - SHARP/SAM/Bison/Workbench

References:

1. Rui Hu, "A fully-implicit high-order system thermal-hydraulics model for advanced non-LWR safety analyses," *Annals of Nuclear Energy*, Vol. 101, 174–181, (2017).
2. R. Hu and Y. Yu, "A Computationally Efficient Method for Full-Core Conjugate Heat Transfer Modeling of Sodium Fast Reactors," *Nuclear Engineering and Design*, Vol. 308, 182-193, (2016).
3. R. Hu and T. Sumner, "Benchmark Simulations of the Thermal-Hydraulic Responses during EBR-II Inherent Safety Tests using SAM", *Proceedings of ICAPP'16*, San Francisco CA, April 17-20, 2016.
4. T.H. Fanning and R. Hu, "Coupling the System Analysis Module with SAS4A/SASSYS-1", ANL/NE-16/22, ANL-ART-74, Argonne National Laboratory, September 2016.
5. R. Hu, T.H. Fanning, T. Sumner, Y. Yu, "Status Report on NEAMS System Analysis Module Development," ANL/NE-15/41, Argonne National Laboratory, December 2015.

Thank you for your attention!
&
Questions?



The submitted manuscript has been created by UChicago Argonne, LLC, Operator of Argonne National Laboratory (“Argonne”). Argonne, a U.S. Department of Energy Office of Science laboratory, is operated under Contract No. DE-AC02-06CH11357. The U.S. Government retains for itself, and others acting on its behalf, a paid-up nonexclusive, irrevocable worldwide license in said article to reproduce, prepare derivative works, distribute copies to the public, and perform publicly and display publicly, by or on behalf of the Government.



BACKUP SLIDES: Glance of Input File – Tool Usability

[Components]

...

[./core]

type = HexLattice
position = '0 0 0'
orientation = '0 0
n_side = 2
assem_pitch = 0.
assem_Dft = 0.13

assem_layout = 'l
F'

radial_power_pea

ref_hs = referenc
ref_duct = duct_w

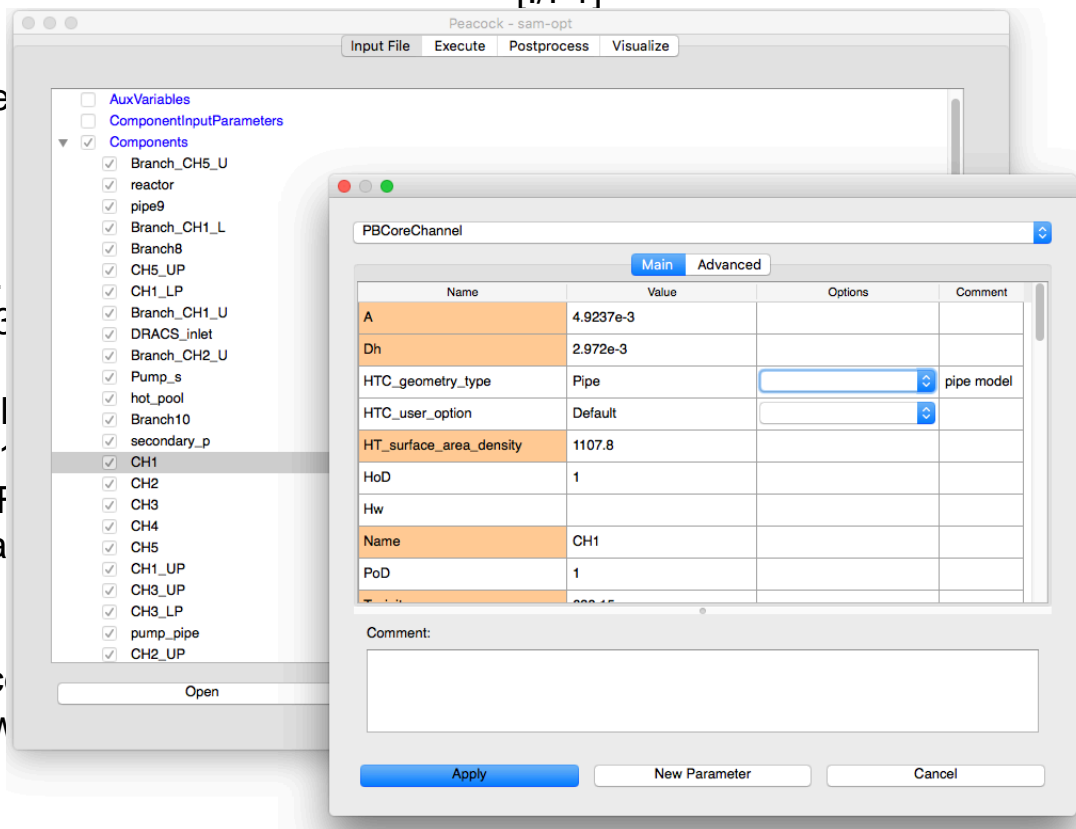
[./]

[]

[ComponentInputParameters]

[./F1]

parameters



beta = 0.01

[./]

...

[]



Closure Model Enhancements

- Review, select, and implement a set of thermal-hydraulic closure models
 - Focused on heat transfer and friction correlations
 - Sodium, simple fluoride salt and air EOS models
- Interactions with IAEA NAPRO Coordinated Research Project
- Selection criteria and points of consideration:
 - Fluid type (sodium) and flow geometry examined
 - Applicable flow regime and geometric ranges
 - Uncertainty of correlation
 - Model complexity / user-accessibility
 - Popularity amongst system code users
 - State-of-the-art investigation
- Verified with unit test problems
- Initial study on the effects of closure models in SFR safety analysis



SAM V&V Plan

- Phenomena Identification
- Software Verification
 - Regression, Benchmark Tests, and Bug Reporting
 - Documentation
 - Verification with Analytical Benchmarks
 - Verification with Code-to-Code Comparison
- Software Validation
 - Unit, Component, Subsystem, System, Integral tests
 - Validation Matrix and Gaps
 - Schedule and Priorities



ANL/NE-14/14

TEST MATRIX FOR CODE VERIFICATION															
	Facility 1	Facility 2	Facility 3	Facility 4	Facility 5	Facility 6	Facility 7	Facility 8	Facility 9	Facility 10	Facility 11	Facility 12	EBR-II	FFTF	SFR-3
Number of problems:	n	n	n	n	n	n	n	n	n	n	n	n	n	n	n
BASIC PHYSICOLOGICAL MODELS															
IC factor form losses	X	X	X					X					X	X	X
Wall surface roughness	X	X	X								X		X	X	X
Wall drag friction		X	X	X	X								X	X	X
Alighted flow area change form losses						X	X	X	X			X	X	X	X
Grossly head term in median equation	X	X	X		X	X		X		X			X	X	X
2D solid and seal conduction					X	X							X	X	X
Fluid conduction		X		X	X							X		X	X
Convection heat transfer		X	X	X	X					X	X		X	X	X
Fuel (rod/gap/rod) models		X		X	X		X						X	X	X
Constant power source				X	X										X
Adiabatic inner surface					X								X	X	X
Materials properties data				X									X	X	X
Parallel channel flow							X						X	X	X
Flow coupling	X		X							X			X	X	X
Multiple connections	X		X				X			X			X	X	X
Well heat transfer for 0-D components						X				X			X	X	X
Heat generation in 0-D components						X				X			X	X	X
Reactivity feedback							X			X			X	X	X
Wire spacer friction							X			X			X	X	X
Minimum convection		X	X	X									X	X	X
Subcooled mixing							X						X	X	X
Powered and unpowered heat structures													X	X	X
BOUNDARY CONDITIONS															
No flow BC								X	X				X	X	X
Inlet velocity function/table BC									X	X			X	X	X
Mass flow function/table BC											X		X	X	X
Constant atm pressure BC										X			X	X	X
Inlet/Outlet pressure function/table BC													X	X	X
Inlet temperature function/table BC													X	X	X
Constant power					X								X	X	X
Power vs time										X			X	X	X
Steady state asymptotic calculation										X			X	X	X
Scanned power vs time													X	X	X
TYPES OF CALCULATIONS															
Single phase flow transient		X	X	X		X		X	X	X	X		X	X	X
Transient heating/cooldown			X	X		X				X			X	X	X
Pump coast down										X			X	X	X
Thermal stratification			X							X			X	X	X
Transition to natural circulation					X					X			X	X	X
Core flow redistribution			X			X							X	X	X
Subcooled flow redistribution							X						X	X	X
Numerical convergence								X		X			X	X	X
Restart calculation								X					X	X	X
Calculation reproducibility								X					X	X	X
All control signal parameters										X			X	X	X
COMPONENTS USED															
Core/Heat Exchanger	X	X	X	X	X	X	X	X	X	X	X	X	X	X	X
Coupled Heat Structure	X	X		X				X					X	X	X
Pipe													X	X	X
Heat Exchanger													X	X	X
Core Channel				X						X			X	X	X
Ducted Core Channel													X	X	X
Pyrolytic Channel				X									X	X	X
Fuel Assembly				X									X	X	X
Ducted Fuel Assembly				X									X	X	X
Reactor Core													X	X	X
Branch	X	X	X	X	X	X	X		X				X	X	X
Single Junction	X							X	X	X	X	X	X	X	X
Pump	X		X	X	X		X		X		X	X	X	X	X
Valve/Orifice/branch		X		X		X							X	X	X
Control Valve										X			X	X	X
Liquid Volume			X			X					X		X	X	X
TDV	X	X	X	X	X	X	X	X	X	X	X	X	X	X	X
TEV	X	X	X	X	X	X	X	X	X	X	X	X	X	X	X
Coupled TDV													X	X	X

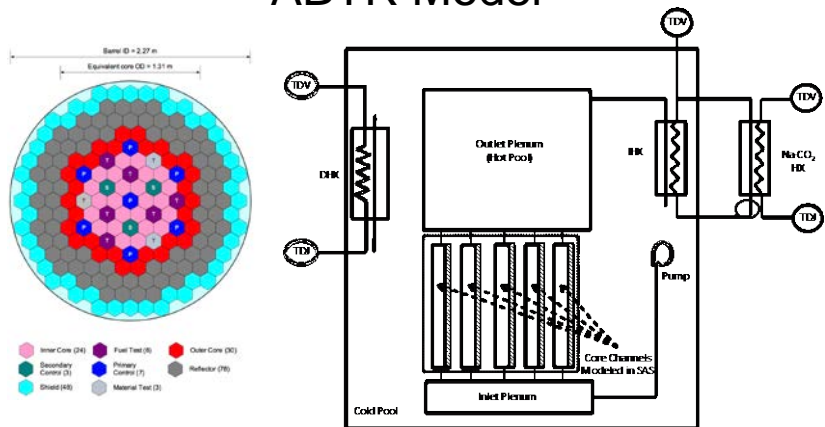


U.S. DEPARTMENT OF
ENERGY

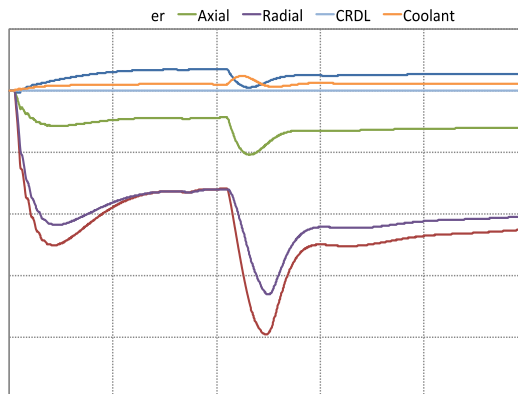
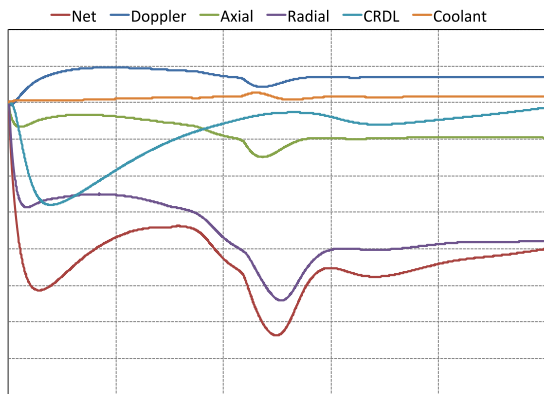
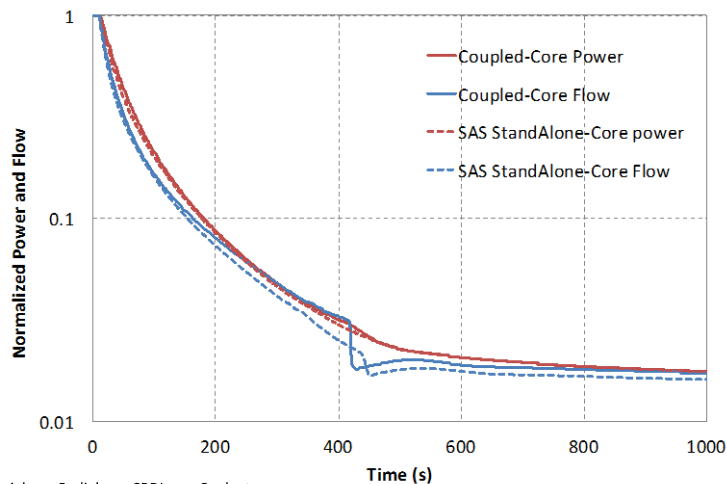
Nuclear Energy

Demonstration of SAS/SAM Coupling

ABTR Model



Power and Flow during ABTR ULOF



Reactivity Feedback Effects