



# Technology Maturation – Conclusions and Future Work Ideas

**Holly Trelue**  
**Technical Area Lead**

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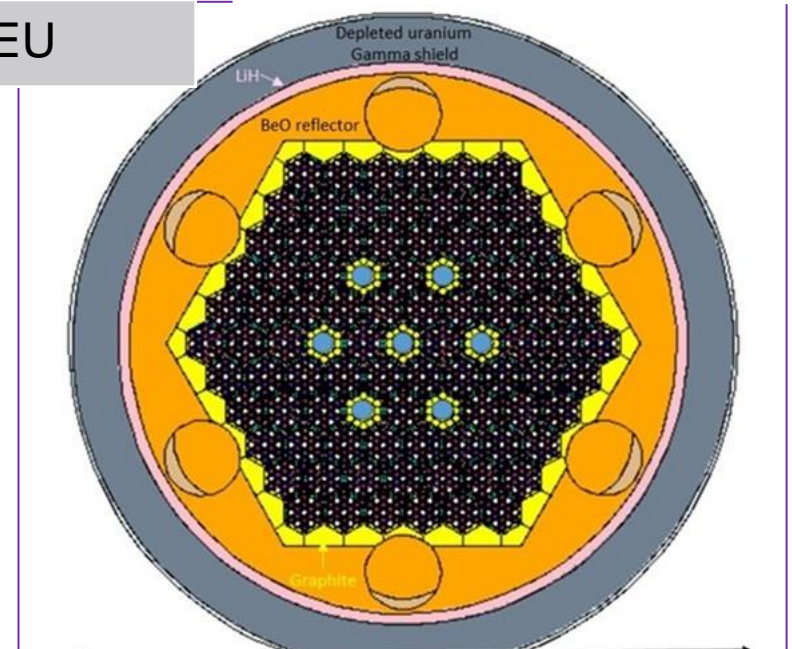
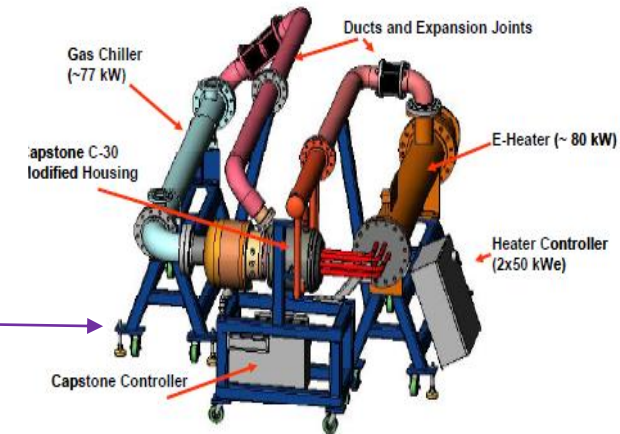
# In summary

- Technology maturation is a vital part of the microreactor program to help numerous vendors achieve higher TRLs and successfully deploy microreactors.
- Work on 37 heat pipe test article and PIE of  $YH_x$  will yield valuable information for microreactors and are a focal point of FY22.
- Embedded sensors and other measurements at MAGNET are also important.
- More technology maturation is required for microreactor building and deployment though.
- Structural material, autonomous control, legacy fuel and other activities may be examined soon, but the list of needs does not end there.

# Needs

Feature	Commercial Reactors	Microreactors
Cooling	Water	Heat pipes, gas
Control	Soluble boron	Control drums
Electricity Generation	Steam Generator /Turbine	Heat Exchanger/ Power Conversion
Moderator	Water	Hydrides/Graphite?
Enrichment	<5% UO <sub>2</sub>	<20% HALEU

- Shielding, reflectors, and control material such as drums or burnable poison need to be assessed along with safeguards and other topics in conjunction with other programs (i.e. NEAMS, MPACT, NEET).
- Power conversion unit (PCU) and other technology such as gas coolant are important in addition to enhanced measuring techniques at MAGNET.



# Additional Workscope Planned for FY22 (assuming \$25M and end of CR)

High Temperature Moderator	<ul style="list-style-type: none"> <li>- Encapsulation and containment of YH</li> <li>- Advanced Coatings</li> </ul>
Instrumentation and Sensors	- Microreactor Autonomous Control System for graded response of critical systems
Fuel Qualification	Complete microreactor activities on legacy fuel qualification of metallic fuels.
Heat Transfer	<ul style="list-style-type: none"> <li>- Support PCU integration</li> <li>- Fabricate gas-cooled test article</li> </ul>
Instrumentation and Sensors	- Acoustic Sensors, Improve DIC
Structural Materials	Refractory metals: Perform initial investigations of advanced materials and manufacturing for microreactors such as TZM and gap analysis on code qualifying

# Potential Future Work Scope

- Fabricate graphite/based heat pipe test article
- Integrate power conversion unit with test articles/heat exchanger for thermal energy conversion
- Complete PIE and other analyses of  $YH_x$  samples irradiated in ATR
- Fabricate and test encapsulated sample of  $YH_x$  for hydrogen permeation for long lifetime operation
- Examine other types of moderator material (Be, graphite,  $ZrD_2$ )
- Complete evaluation of burnable poisons, reflectors, control drums, and/or shielding material options for microreactors
- Evaluate results of instrumentation and sensor measurements of test articles at MAGNET
- Develop acoustic or other structural health monitoring techniques for microreactors (embedded fiber optic sensors, piezoelectric sensors, traditional bonded strain gauges, DC potential drop, or non-contact techniques)
- Extend distributed temperature and strain sensing capabilities from SPHERE to MAGNET test articles
- Fabricate metal refractory test articles to examine structural material integrity
- Test safeguards instrumentation at MAGNET or MARVEL



# Determining the Effects of Neutron Irradiation on the Structural Integrity of Additively Manufactured Heat Exchangers for Very Small Modular Reactor Applications

*John Gahl<sup>1</sup>, Scott Thompson<sup>2</sup>, **Bart Prorok<sup>3</sup>**, Valentina O'Donnell<sup>1</sup>,  
Mohanish Andurkar<sup>2</sup>, Tahmina Keya<sup>3</sup>, Ashley Romans<sup>3</sup>, Greyson Harvill<sup>3</sup>*

<sup>1</sup>University of Missouri Research Reactor (MURR), University of Missouri

<sup>2</sup>Alan Levin Department of Mechanical and Nuclear Engineering, Kansas State University

<sup>3</sup>Department of Materials Engineering, Auburn University

**MICROREACTOR PROGRAM REVIEW, MARCH 2022**



**MURR<sup>®</sup>**  
Matters...

**KANSAS STATE**  
UNIVERSITY

**AU** AUBURN  
UNIVERSITY

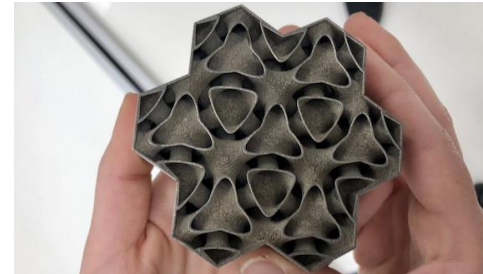
 **MRP** Microreactor  
Program

# overview

- Nuclear applications
- Project Objectives
- Materials and Production Method
- Radiation Experiments
- Experimental Results
- Summary, Conclusions, and Continuing Work

# Additive manufacturing – complex hExs

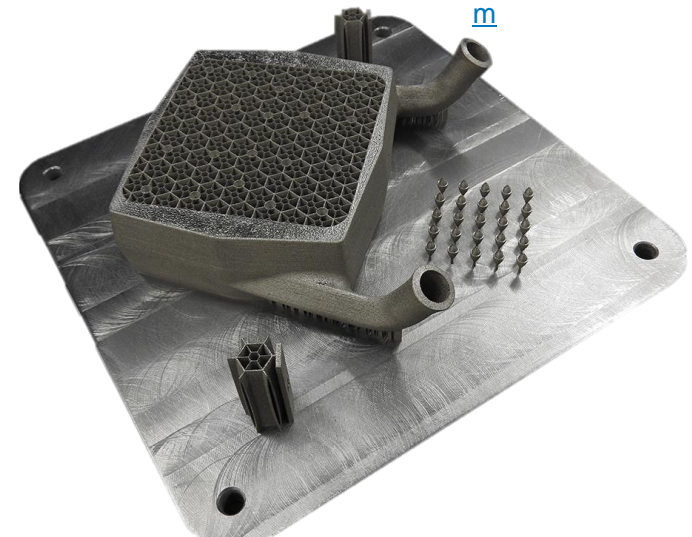
- Additive Manufacturing enables novel geometries and performance
  - part/joint consolidation
  - non-uniform cross-sectioned channels,
  - asymmetric core architecture,
  - fully-circular channels
- Effects of neutron irradiation on AM not well studied yet
  - Nuclear Attenuation
  - AM Processing Characteristics
  - Microstructure / mechanical / corrosion properties
- AM Microstructures differ from wrought microstructures
  - AM microstructures are anisotropic
  - Limited to no thermal-mechanical processing



<https://www.metal-am.com/>



[www.renishaw.com](http://www.renishaw.com)



<https://design-engineering.polimi.it/portfolio/additive-manufacturing-and-heat-exchangers/>



# Project Objectives

**Determine effects of thermal and fast neutron irradiation on additively manufactured (AM) 625 and 718 nickel-based super alloys**

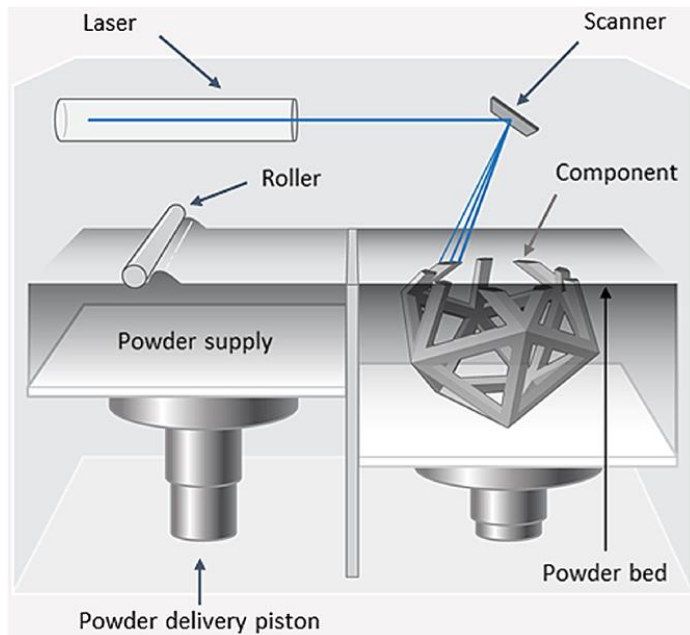
- **Neutron Irradiation Effects**

- Swelling
- Voids
- Frenkel pairs
- Dislocations
- Hardening & Embrittlement
- Impurity formation (hydrogen, helium)

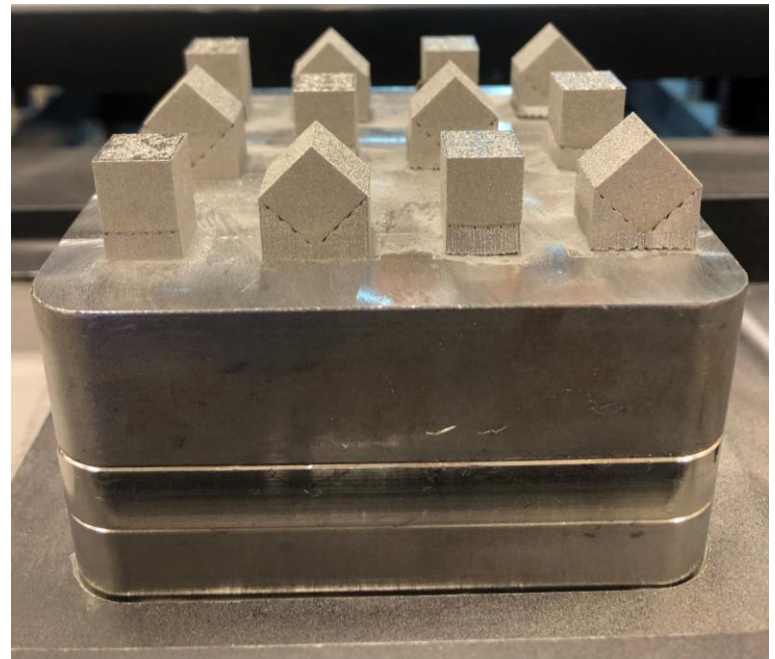
- **Microstructure/mechanical properties**

- Micro/nano hardness
- As-printed microstructure
- Heat-treatment effects
- AM built angle effects

# Laser Powder bed fusion (L-PBF)



O'Brien, *Optical Engineering* **58**(1), 010801 (2019)



# Heat-treatment schedules

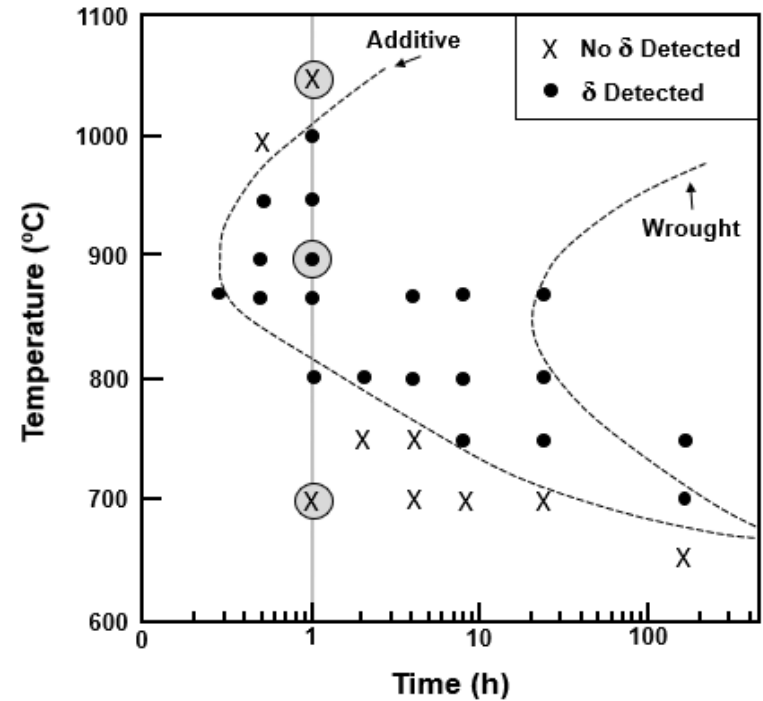
## Heat treatment schedules for this study

- As-built (no heat treatment)
- 700 °C - 1 hour
- 900 °C - 1 hour
- 1050 °C - 1 hour

## Precipitates formed

- $\gamma''$ - contributes to hardening effect in microstructure
- $\delta$ - Needle and plate shaped precipitate (deleterious)- makes part brittle
- Both precipitates are rich in Niobium
- 1050 °C -1h was carried out to dissolve all precipitates back in Ni-Cr matrix.
- Final microstructure resembled wrought IN625 microstructure.

Heat treatments were performed on samples of both build orientations



<https://doi.org/10.1007/s11661-018-4643-y>

# Radiation Experiments

## MURR

### Full-Spectrum Neutron

- 10 MW reactor at MURR
- 310 hours
- Neutron flux =  $6.61 \times 10^{13}$  neutrons/cm<sup>2</sup>/s
- Neutron fluence =  $7.37 \times 10^{19}$  neutrons/cm<sup>2</sup>

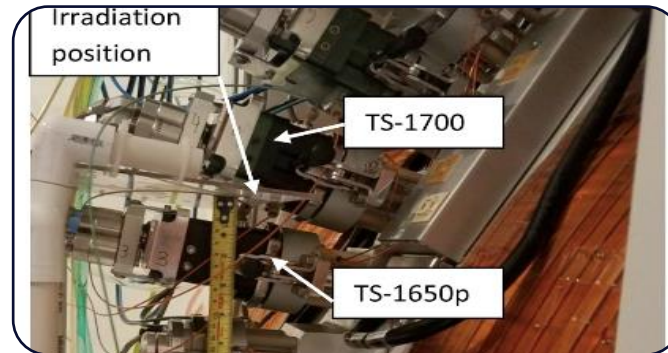
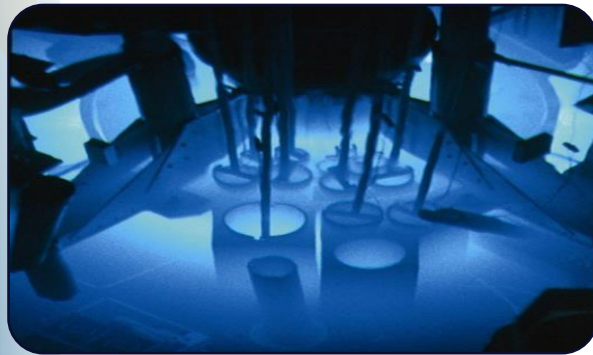
## 16 MeV Cyclotron

### (Fast) Neutron

- Neutrons from PET isotope production  $^{18}\text{O}(p, n)^{18}\text{F}$  reaction
- 25-week experiment
- Neutron fluence =  $9.08 \times 10^{15}$  neutrons/cm<sup>2</sup>
- Low fluence, but low activity

### Proton

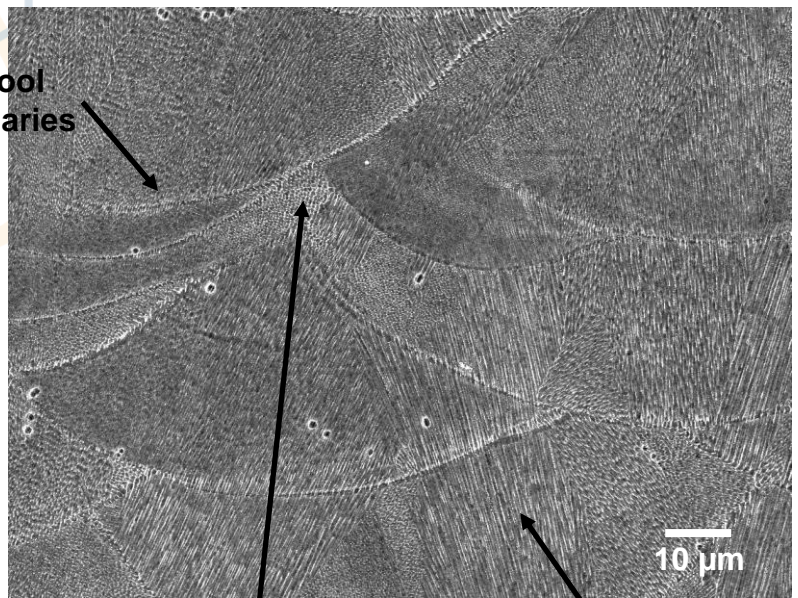
- 80  $\mu\text{A}$  BEAM
- Currently investigating the methodology



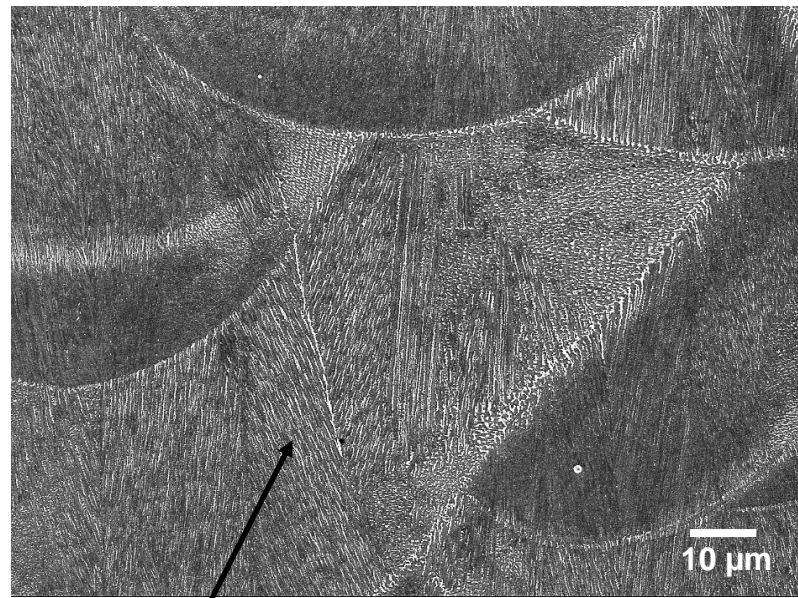
# MICROSTRUCTURE

## As-Built Microstructure

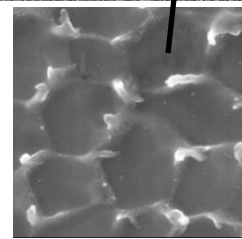
Melt-Pool  
Boundaries



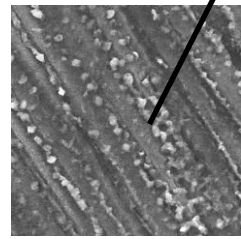
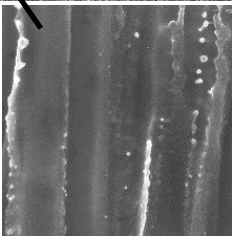
## 700°C Microstructure



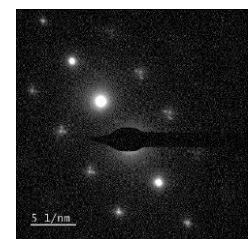
↑ Build  
Direction



Dendritic  
Solidification  
+  
Interdendritic  
Chemical  
Separation

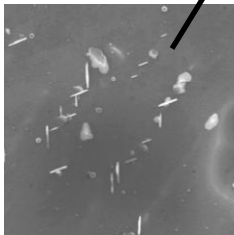
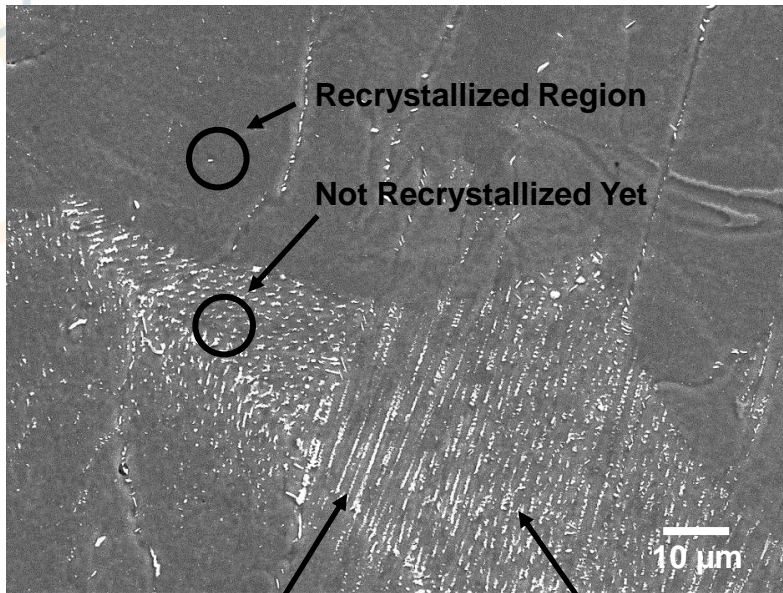


$\gamma''$  ppt  
Formation in  
Interdendritic  
Region

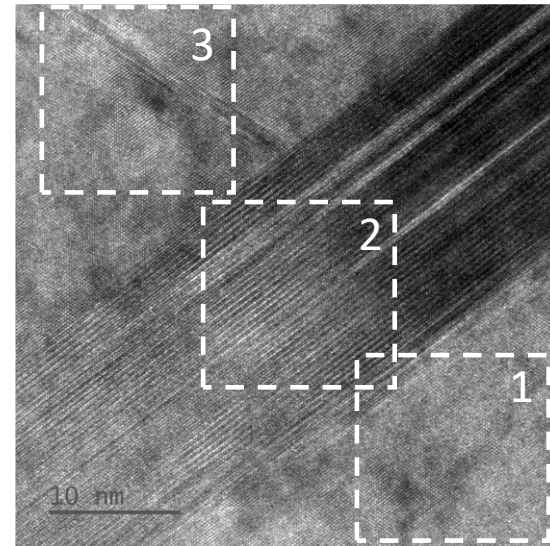
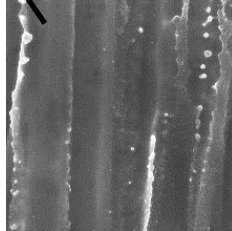


# MICROSTRUCTURE

## 900°C Microstructure



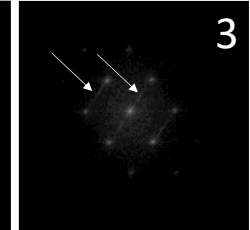
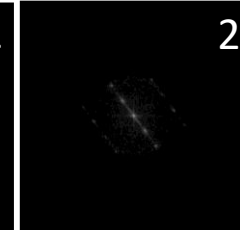
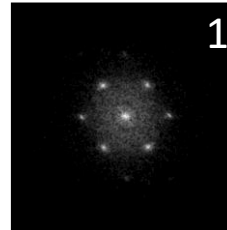
$\delta$  ppt  
Formation in  
Interdendriti  
c Region



Matrix  $\langle 110 \rangle$

$\delta$  ppt

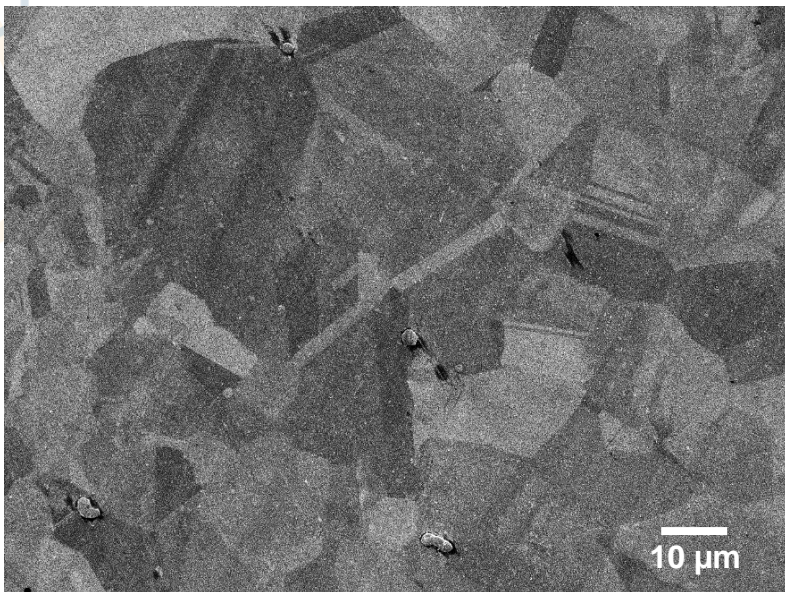
Dislocation



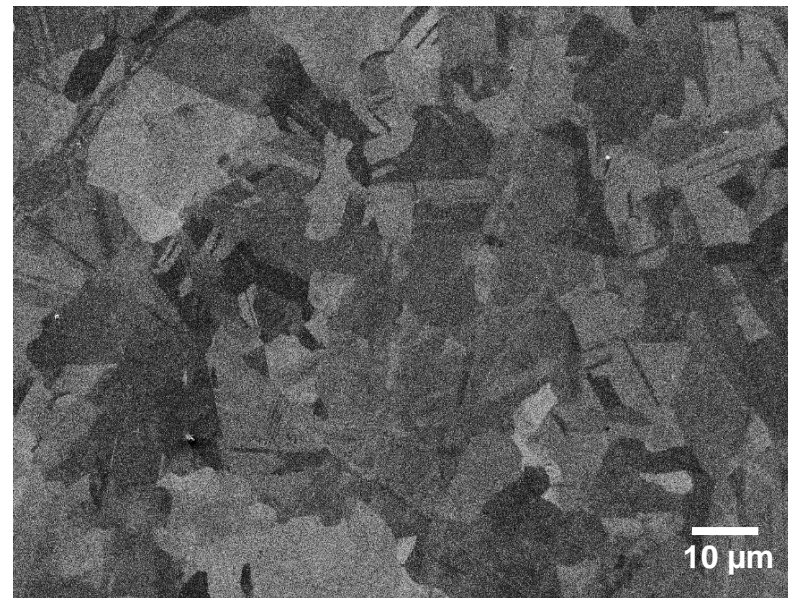
↑ Build  
Direction

# MICROSTRUCTURE

As-Built Microstructure



700°C Microstructure



Completely  
Recrystallized

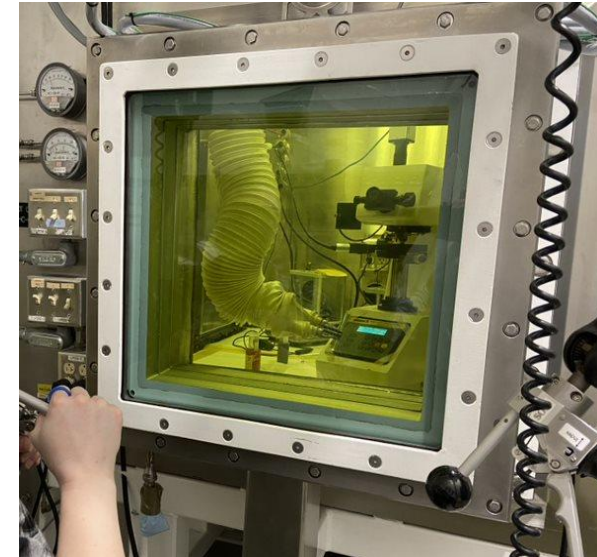
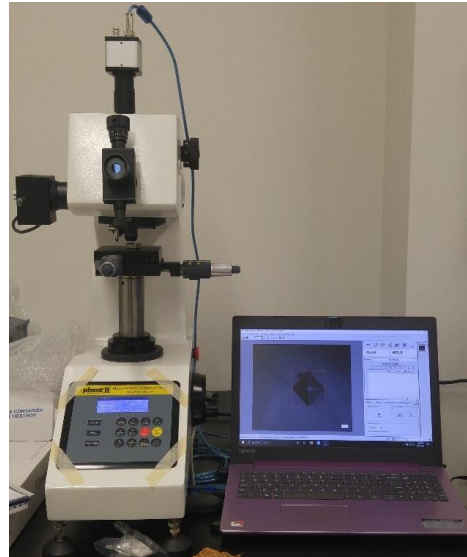
ppts  
Dissolved

↑ Build  
Direction

# Microhardness measurements

## Vickers Microhardness

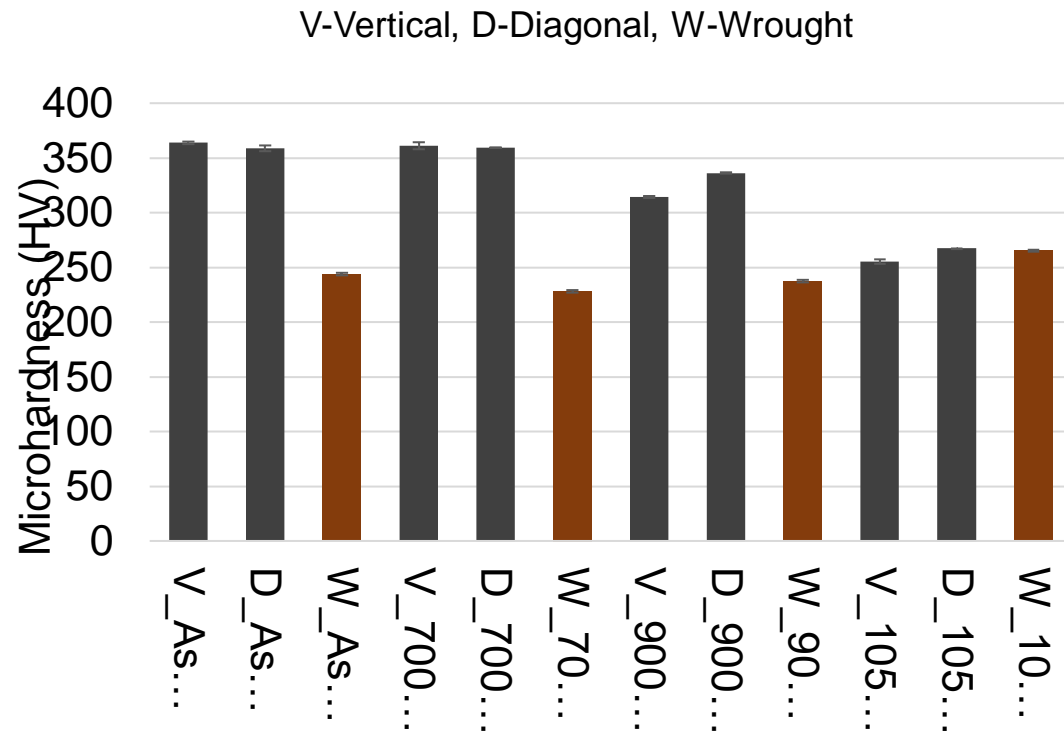
- Phase II 900-391D
- Load = 1 kgf (9.8 N)
- Dwell time = 15 secs
- Hot cell and manipulator arms for radioactive samples
- Hardness measurements pre- and post- irradiation to track radiation-induced changes





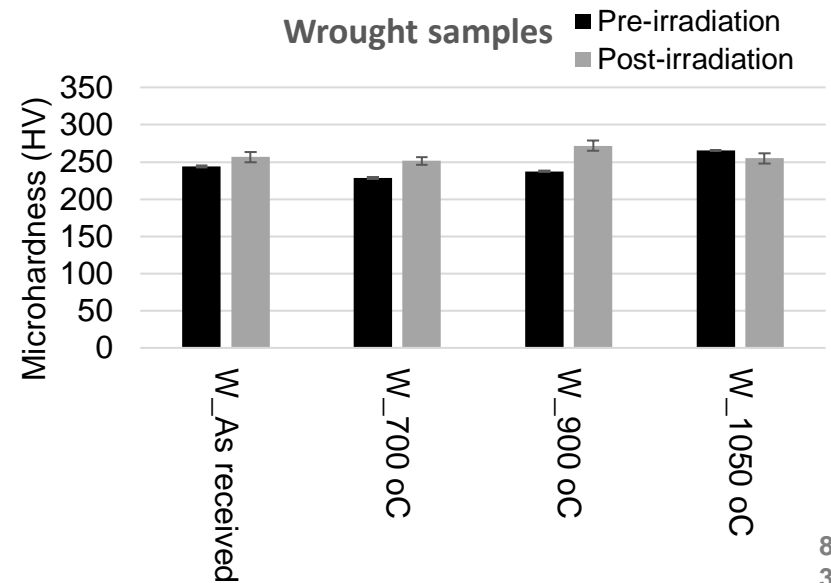
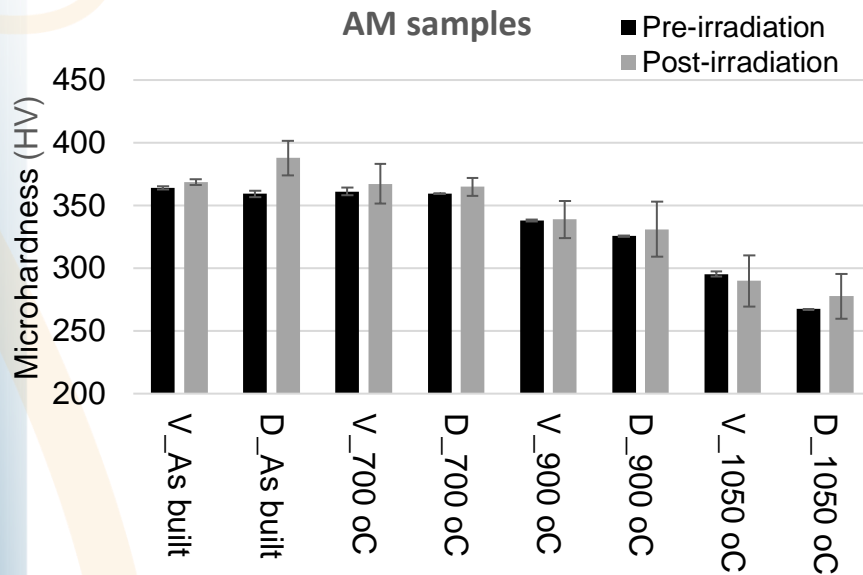
# Microhardness of L-PBF and wrought in625

- As-built Vert and Diag showed minimal variation in microhardness.
  - This shows built directional independence.
- Microhardness decreased as heat treatment temperature increased
- 1050 °C AM samples display similar microhardness to wrought Inconel 625
  - This indicates the microstructures are similar



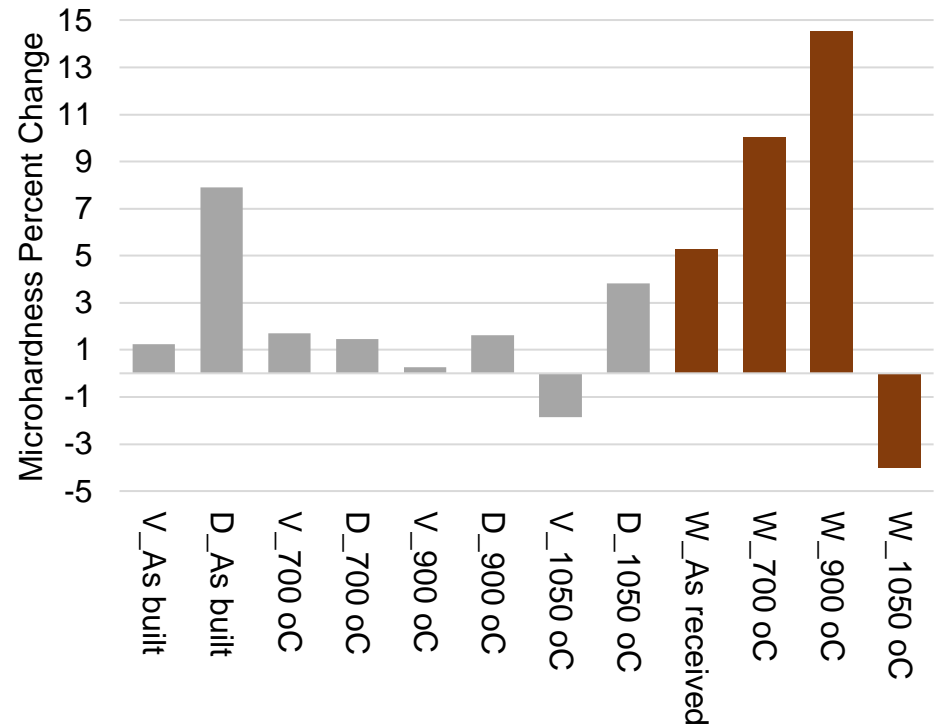
# Full-spectrum neutron irradiation

- Thermal neutron irradiation induced embrittlement/hardening in AM and wrought IN625
- 1050 °C AM vertical and wrought showed slight radiation softening phenomenon
- Preliminary results show AM IN625 to be more resistant to radiation hardening than wrought IN625



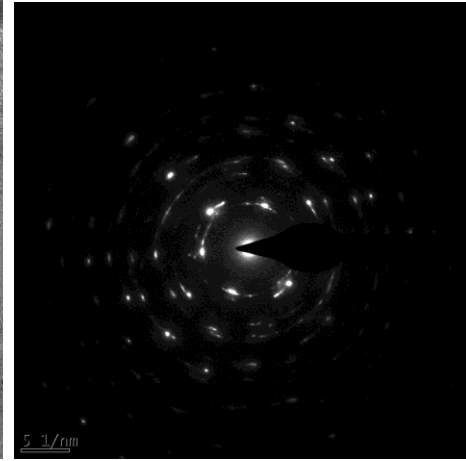
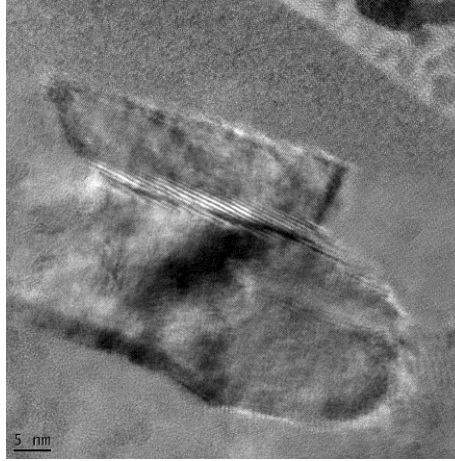
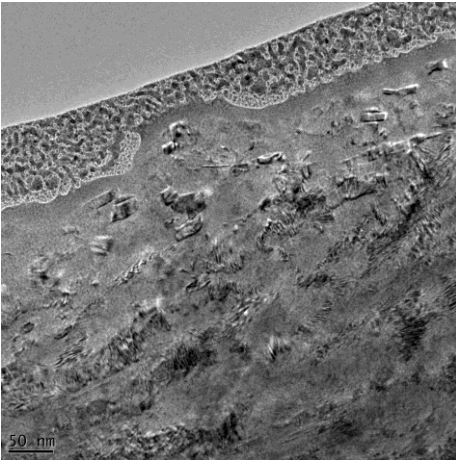
# Full spectrum neutron irradiation

- Vertically printed AM samples hardened 0.4% on average
- Diagonally printed AM samples hardened 3.8% on average
- Conventionally wrought samples hardened 6.1% on average

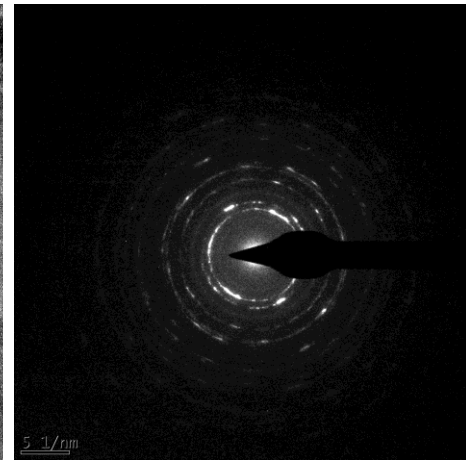
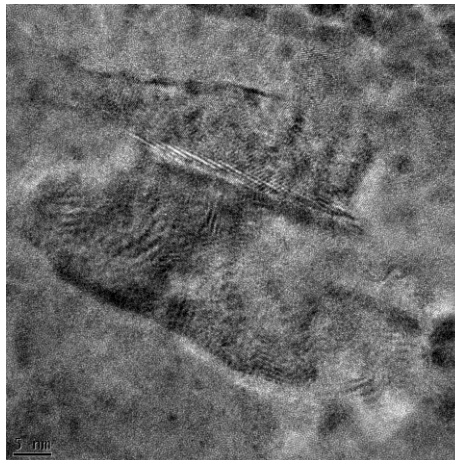
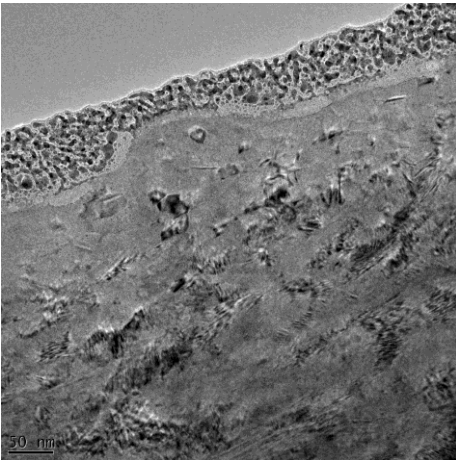


# FAST NEUTRON IRRADIATION – LAMELLA TECHNIQUE

Pre-irradiation

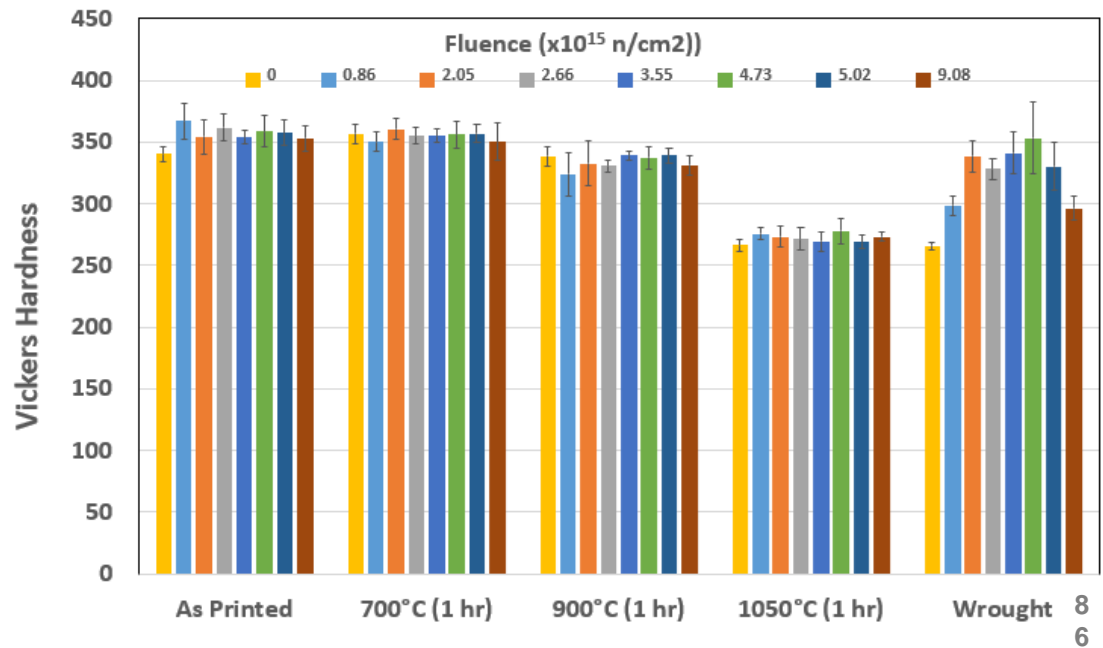


Post-irradiation



# FAST NEUTRON IRRADIATION

- Microhardness was measured at different neutron fluences
- Less variation seen in the microhardness values of AM Inconel 625 compared to wrought Inconel 625
- Lower overall fluence compared to thermal neutron irradiation meaning lower DPA
- No thermal neutrons meaning lower activity



# SUMMARY AND CONCLUSIONS

- Several L-PBF AM IN625 and wrought IN625 samples were irradiated using thermal and fast neutrons.
  - Results showed that L-PBF IN625 samples were more resistant to radiation hardening compared to wrought IN625.
  - Wrought IN625 samples were most prone to radiation hardening with average of 6.1%.
  - Diagonal AM IN625 samples experienced slightly more radiation hardening than vertical AM samples
- L-PBF AM IN625 and wrought IN625 samples were irradiated using fast neutrons
  - Results showed that wrought IN625 hardness values had more variation over the course of the experiment than the AM IN625 samples

# CONTINUING WORK

- Detailed microstructure investigation to thoroughly understand the radiation damage mechanism in L-PBF AM and wrought samples
  - SEM and TEM imaging
  - Investigation of phase contributions, build orientation effects
- L-PBF AM Inconel 718 samples currently undergoing irradiation trials.
- Proton beam experiments are being conducted to supplement full-spectrum neutron irradiation experiments.

# THANKS

## Contact Information

- **John Gahl**
  - Nuclear physics/engineering
  - [gahlj@missouri.edu](mailto:gahlj@missouri.edu)
- **Bart Prorok**
  - Additive manufacturing, materials science
  - [prorobc@auburn.edu](mailto:prorobc@auburn.edu)
- **Scott Thompson**
  - Additive manufacturing, heat transfer
  - [smthompson@ksu.edu](mailto:smthompson@ksu.edu)