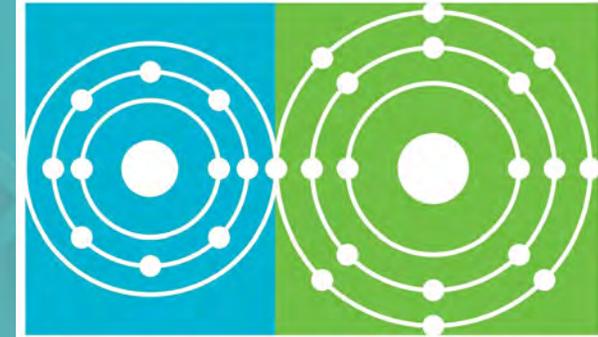


U.S. DEPARTMENT OF  
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**Molten Salt Reactor**  
P R O G R A M

# Beryllium Carbide as Moderator for MSR

Anne Campbell, ORNL



Annual MSR Campaign Review Meeting 16-18 April 2024

# Acknowledgements

- **D. Sulejmanovic, Y. Osetskiy, E. Zarkadoula, A. Willoughby, E. Cakmak, K. Johnson, B. Henry, E. Paxton, S. Fiscor, J. McFarlane, K. Robb – ORNL**
- **Prof. S. Raiman and D. Muzquiz – University of Michigan**
- **D. Holcomb – INL**
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- **Oak Ridge National Laboratory is managed by UT-Battelle, LLC, under contract No. DE-AC05-00OR22725 for the U.S. Department of Energy.**

# Grand Question

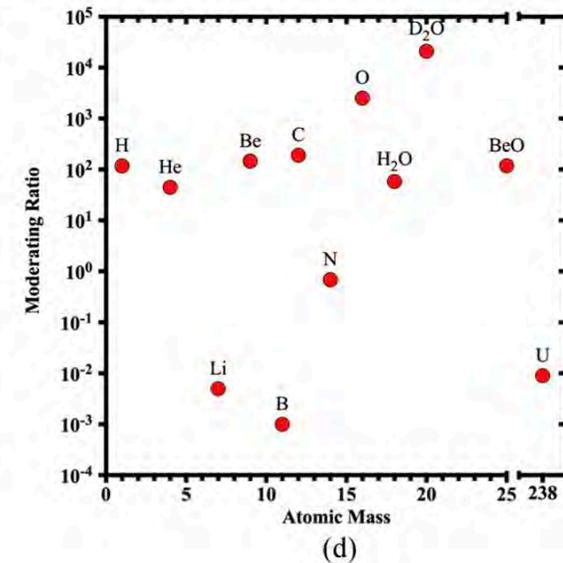
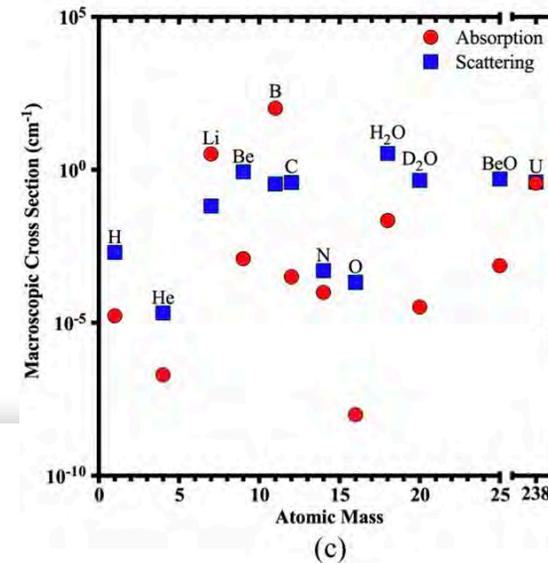
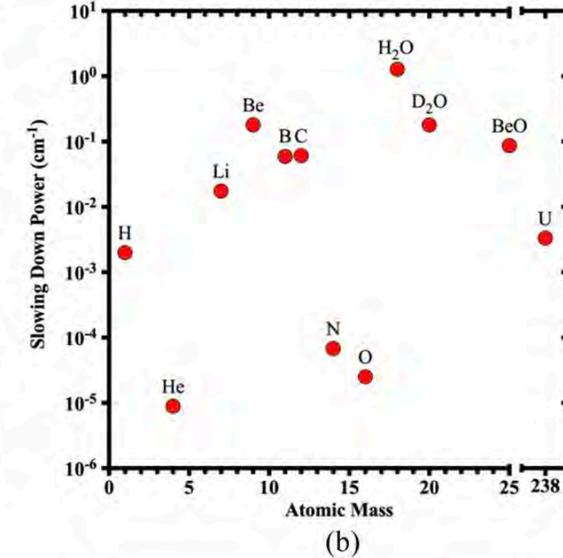
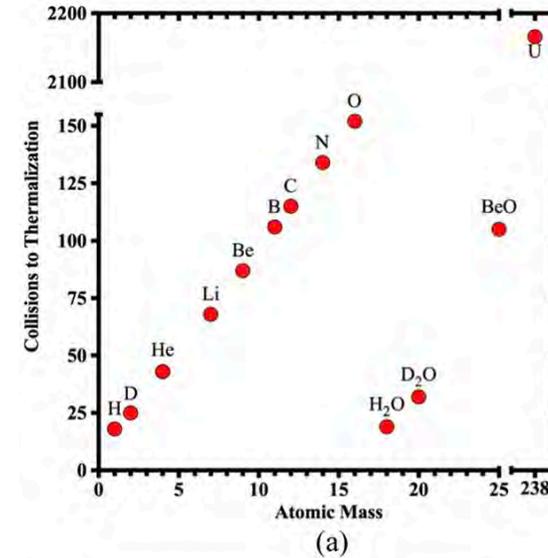
Can beryllium carbide be used in future reactors as a replacement moderator for graphite?

Long-term (10+ years) to answer this question, but can perform preliminary screening

# Why Beryllium Carbide?

Campbell & Burchell Timothy D. (2020). Radiation Effects in Graphite. Comprehensive Nuclear Materials 2nd edition, vol. 3, pp. 398–436

- High moderating efficiency and low absorption cross section
- Be slowing down power  $\sim 2.5x$  > than carbon
- Chemically compatible with coolant salts
- Antifluorite crystal structure – the same crystalline configuration (with anions and cations reversed) as exceptionally radiation damage resistant fluorite type crystals (e.g.,  $UO_2$ )
  - The anti-fluorite crystal ( $Li_2O$ ) has also been shown to have high radiation damage tolerance [1,2]

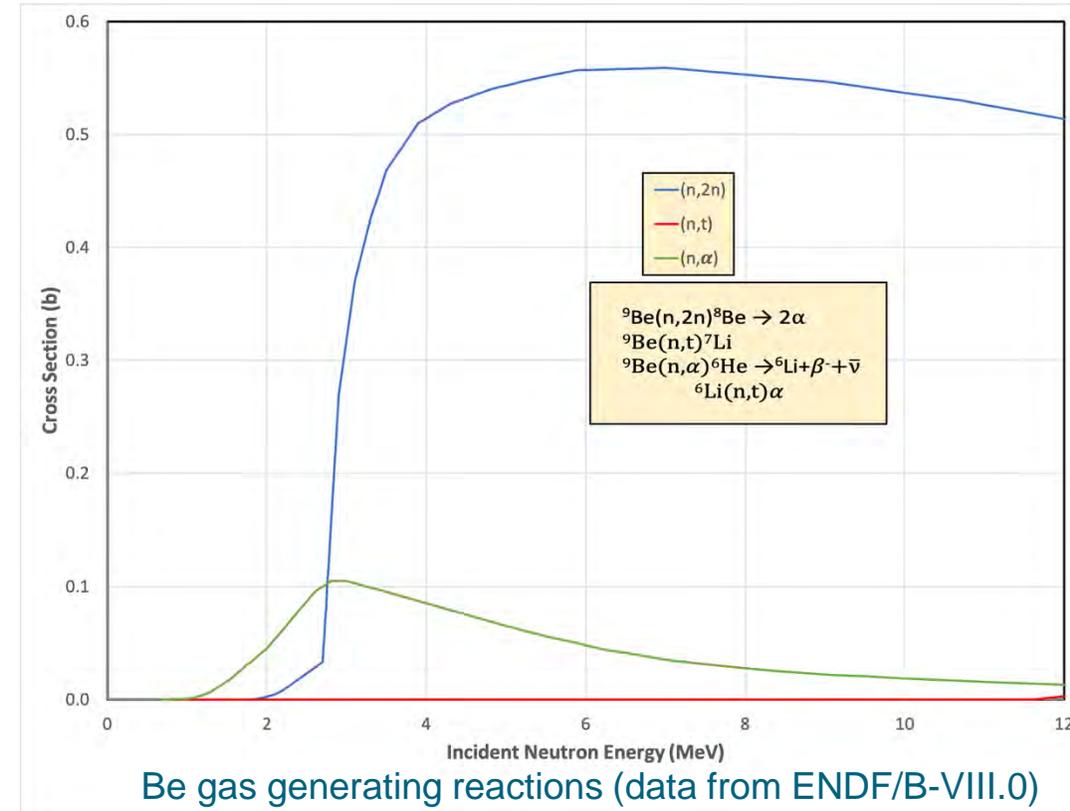


[1] Moriyama et al., *Journal of Nuclear Materials*, **258-263**, (1998) 587-594.

[2] Noda, et al., *Journal of Nuclear Materials*, **123**, (1984) 908-912

# Technical Challenges with Beryllium Carbide

- Long history of graphite as neutron moderators (CP-1, X-10 ~80 years) research and knowledge – only limited low dose studies in  $\text{Be}_2\text{C}$  [1-3]
- $\text{Be}_2\text{C}$  is brittle, vulnerable to thermal stress cracking
  - Can we mitigate brittle nature via fiber reinforcement?
- $\text{Be}_2\text{C}$  is toxic, moisture sensitive, chemically reacts with U
  - Would need a protective layer (NbC)
- $\text{Be}_2\text{C}$  is a methanide (when exposed to H it decomposes into methane)
  - Can this be utilized for tritium management strategy?
  - Methane is easily trapped and doesn't diffuse through metal alloys
- Be does have gas generating reactions with neutrons (He and  $^3\text{H}$ )
  - May be beneficial for fusion systems for  $^3\text{H}$  production



[1] Maya et al., GA-A-17842; (1985)

[2] Marion & Muenzer, SAND--78-0227C, CONF-780622, (1978)

[3] Feldman & Silverman, NAA-SR-114, (1951)

# What are the first steps?

- **Need solid Be<sub>2</sub>C samples – concern is production and processing is export controlled technology**
- **Understand high temperature stability of Be<sub>2</sub>C**
- **Preliminary understanding of irradiation effects in Be<sub>2</sub>C**
- **Degradation behavior when exposed to hydrogen**
- **Understand thermal properties**

# High Temperature Stability Testing

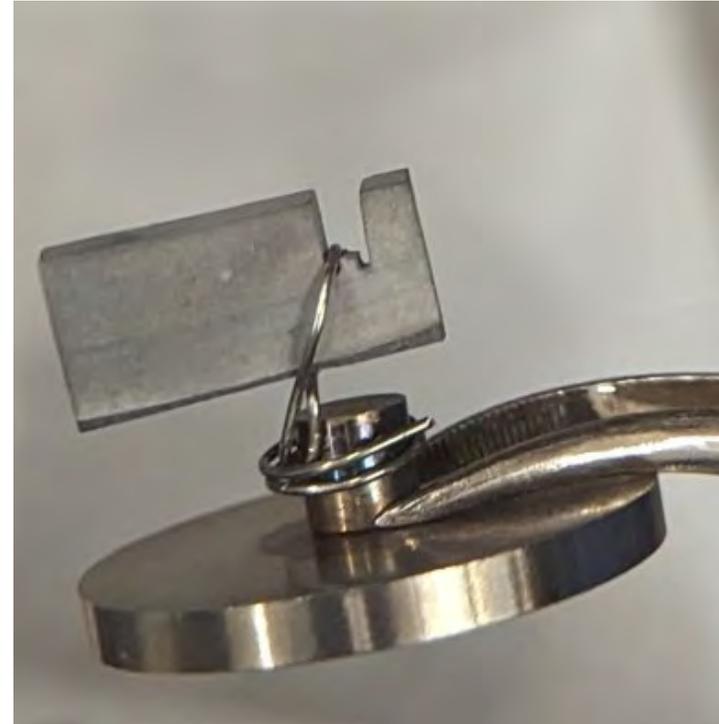
D. Sulejmanovic

A. Willoughby

E. Cakmak

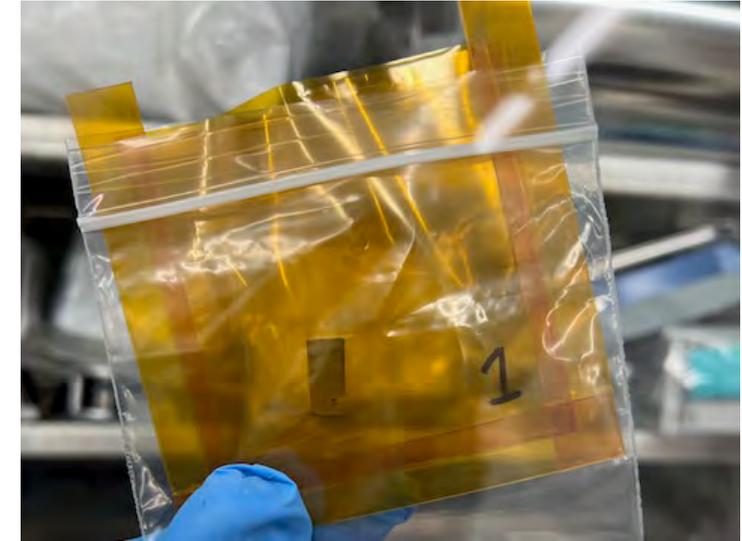
B. Henry

S. Fiscor



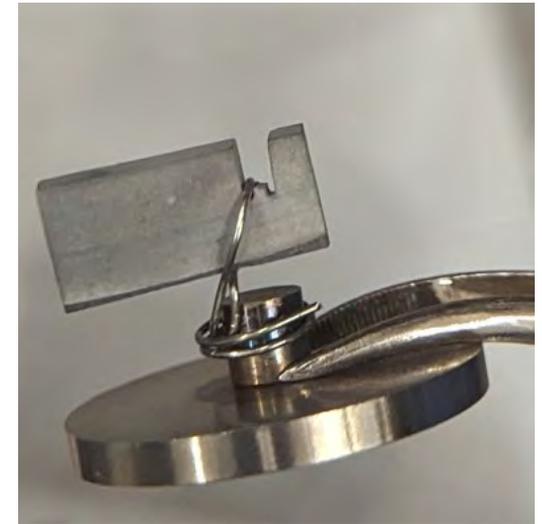
# Phase Composition Measurement

- **Make Kapton packets, load  $\text{Be}_2\text{C}$  into packet and seal shut with 2 pieces of Kapton tape**
- **Panalytical X'pert diffractometer ( $\text{CuK}\alpha$ )**
  - $\theta$ - $2\theta$  setup 20 – 100°  $2\theta$ , with a scan rate of 0.0167 deg/s (~30 minute scan time), 1/4° fixed slits, 1/2° anti-scatter slit, 0.04 soller slits coupled with a 10 mm mask, and zero-background plate was positioned below the specimens to remove any peaks from the metal specimen stage
  - Phase identification used a search match with the Jade software and the ICDD database



# Static Capsule Testing

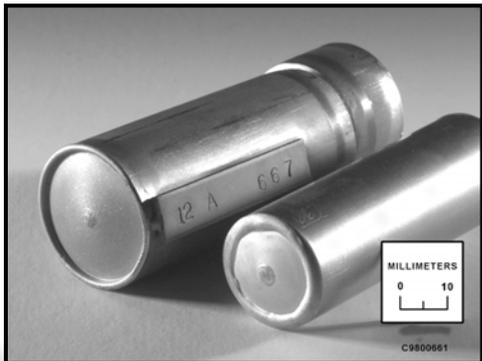
- **Mass specimens**
- **Specimens loaded into 316L stainless steel double-containment capsules**
- **Fill with desired environment (Ar gas)**
- **Electron beam welded shut**
- **Put into box furnace at desired temperature for pre-determined time**



Before exposure



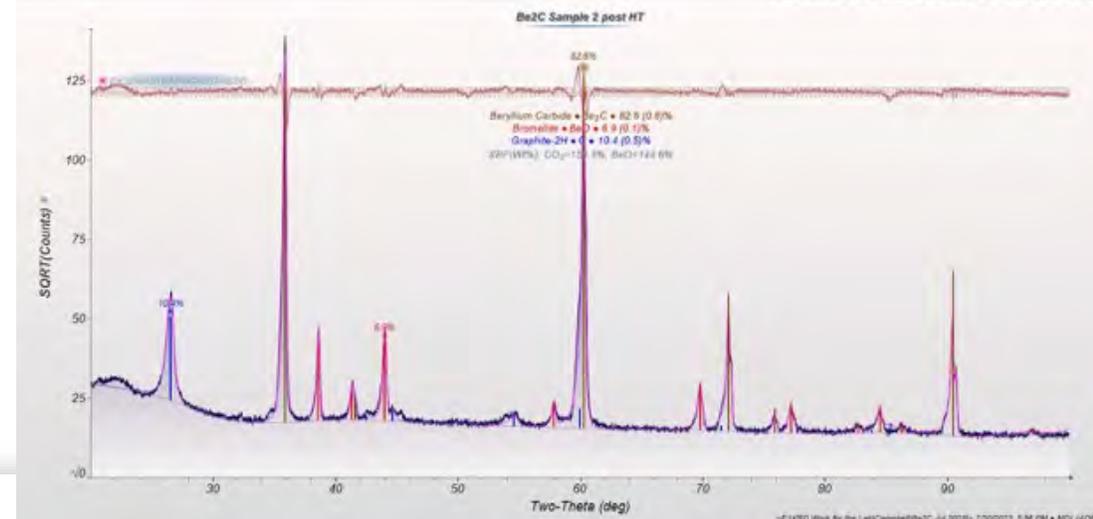
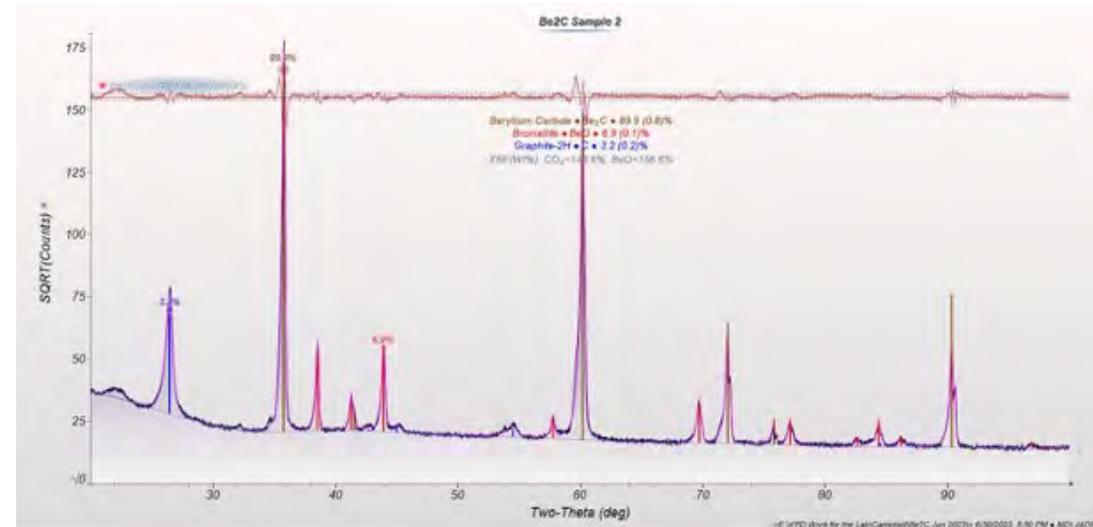
After exposure



Inner and outer capsules  
(courtesy J. Keiser)

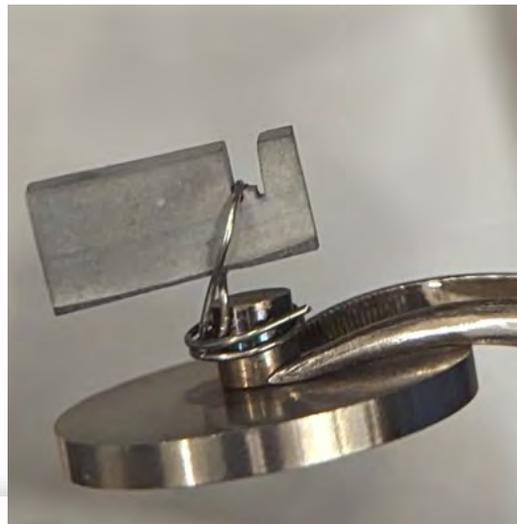
# Post-Exposure Analysis

- Open capsules
- Remeasure specimen mass (mass loss)
- Package in new Kapton packets
- XRD (determine phase composition)



# Summary of Changes 650°C Exposures

Specimen #	Exposure Time	Pre-exposure mass (g)	Post-exposure mass (g)	Mass loss (%)	Be <sub>2</sub> C Phase % before / after	BeO Phase % before / after	Graphite Phase % before / after
1	1 day	0.5596	0.5559	0.66	90.2 / 87.2	7.0 / 7.3	2.1 / 5.5
2	1 week	0.6283	0.6234	0.78	89.9 / 82.6	6.9 / 6.9	3.2 / 10.4
3	2 weeks	0.5960	0.5939	0.35	89.6 / 84.4	7.1 / 7.6	3.3 / 8.0



Before exposure



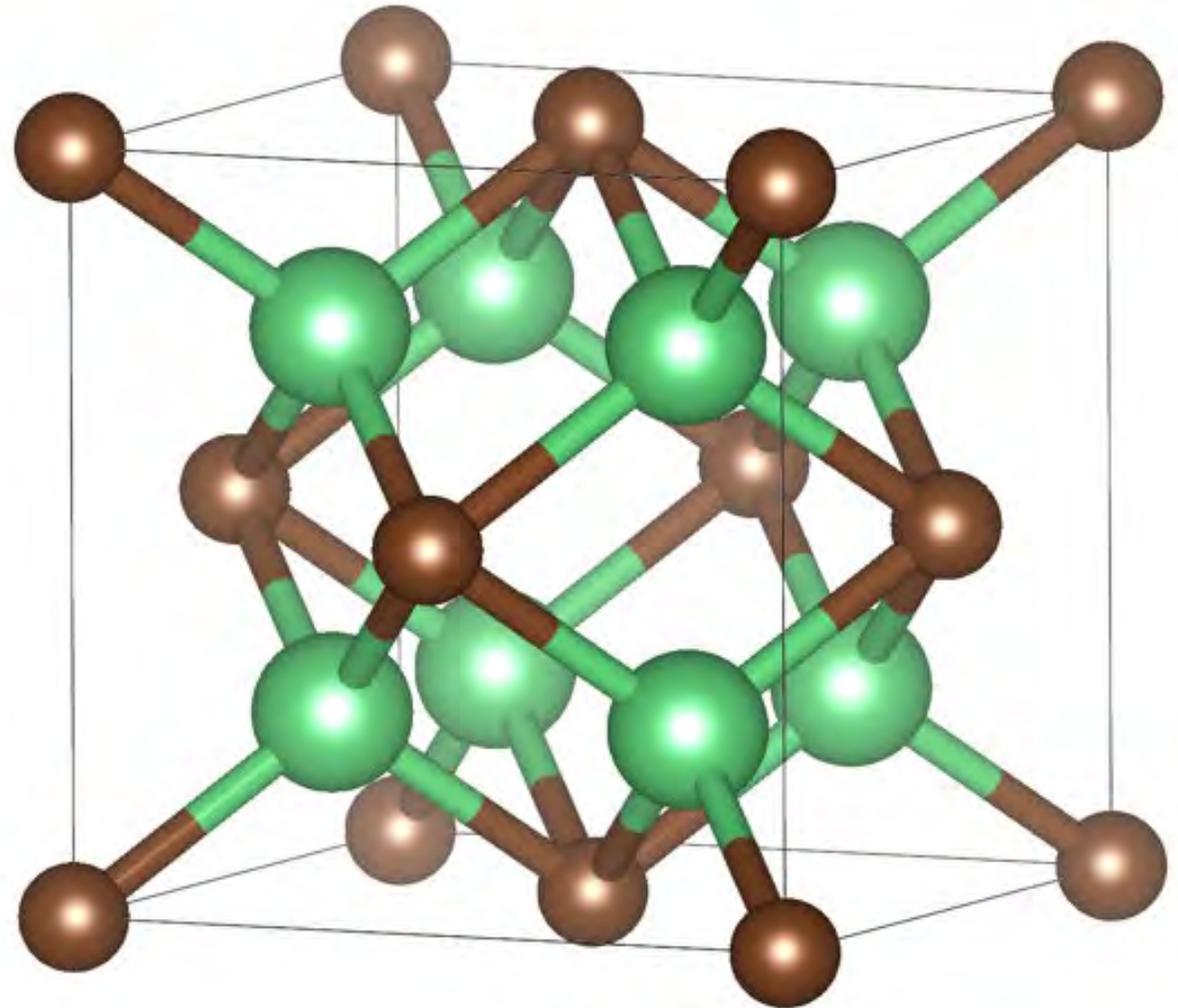
After exposure

Specimens have dull grey finish before exposure. After exposure, all specimens have dark surface (graphite buildup as Be converts to BeO and sloughs off?)

# Modeling Beryllium Carbide

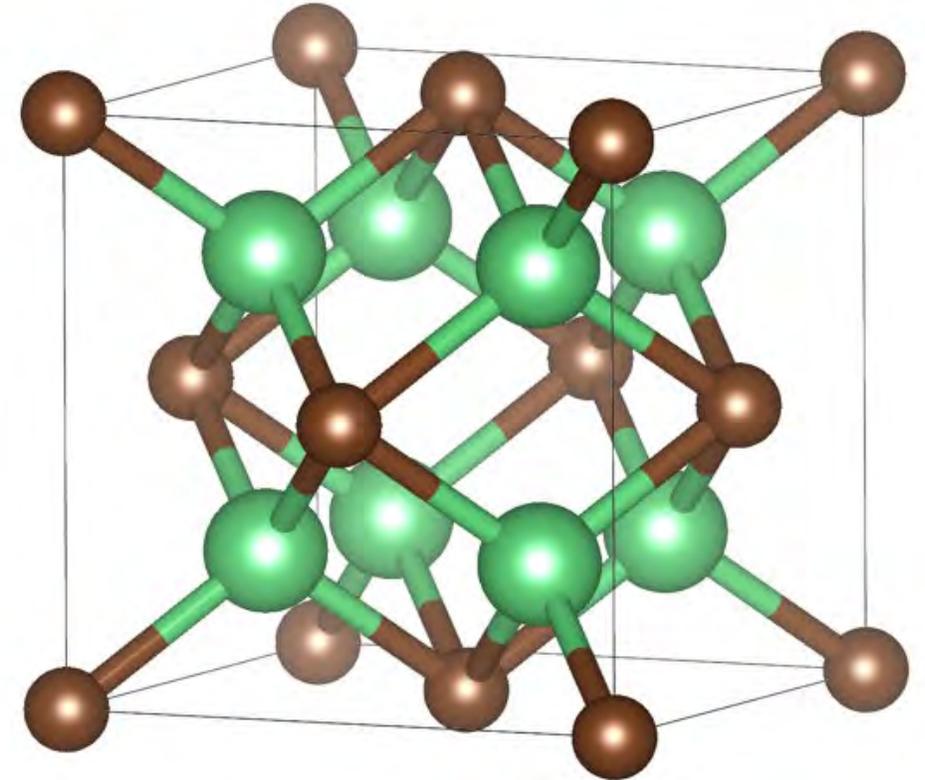
Yuri Osetskiy

Eva Zarkadula



# First principles modeling in $\text{Be}_2\text{C}$

- **DFT modeling in VASP**
- **Supercells of three sizes were used to investigate different effects: 3x3x3 (324 sites), 4x4x4 (768 sites) and 5x5x5 (1500 sites).**
- **Advanced computing facilities: National Energy Research Scientific Computing Center (NERSC) at LBNL and Compute and Data Environment for Science (CADES) at ORNL.**



Atomic structure of anti-fluorite  $\text{Be}_2\text{C}$  structure: Be – green, C – brown

# Band structure

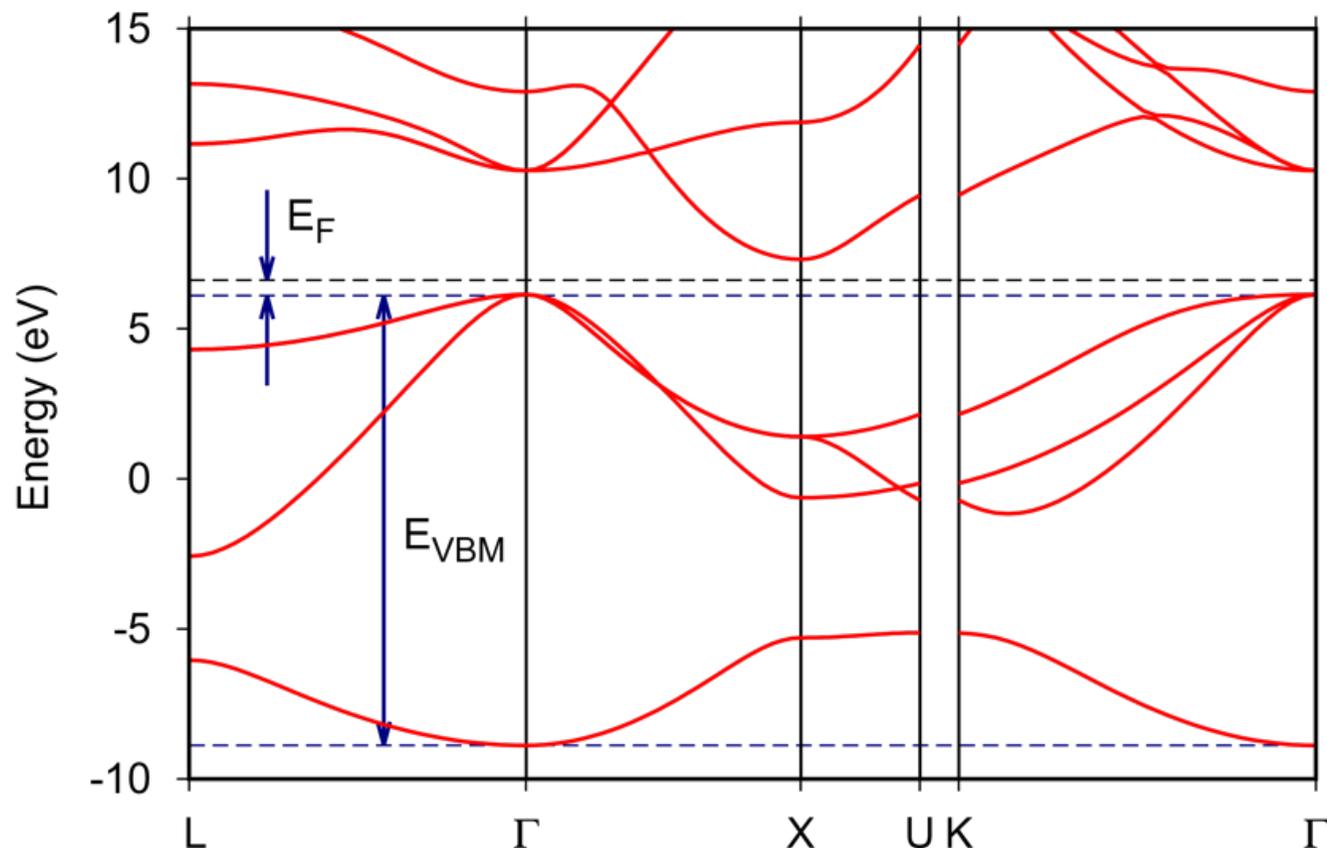
- $\text{Be}_2\text{C}$  is weak semiconductor with relatively narrow band gap:

$$E_g = 1.212 \text{ eV}$$

- Estimated Fermi energy:

$$E_{Fermi} = 6.271 \text{ eV}$$

*Density of states in the perfect  $\text{Be}_2\text{C}$  crystal.  $E_F$  is Fermi energy estimated from the valence band maximum –  $E_{VBM}$ .*



# Density of states and point defects

- **Be<sub>2</sub>C is weak semiconductor with relatively narrow band gap:**

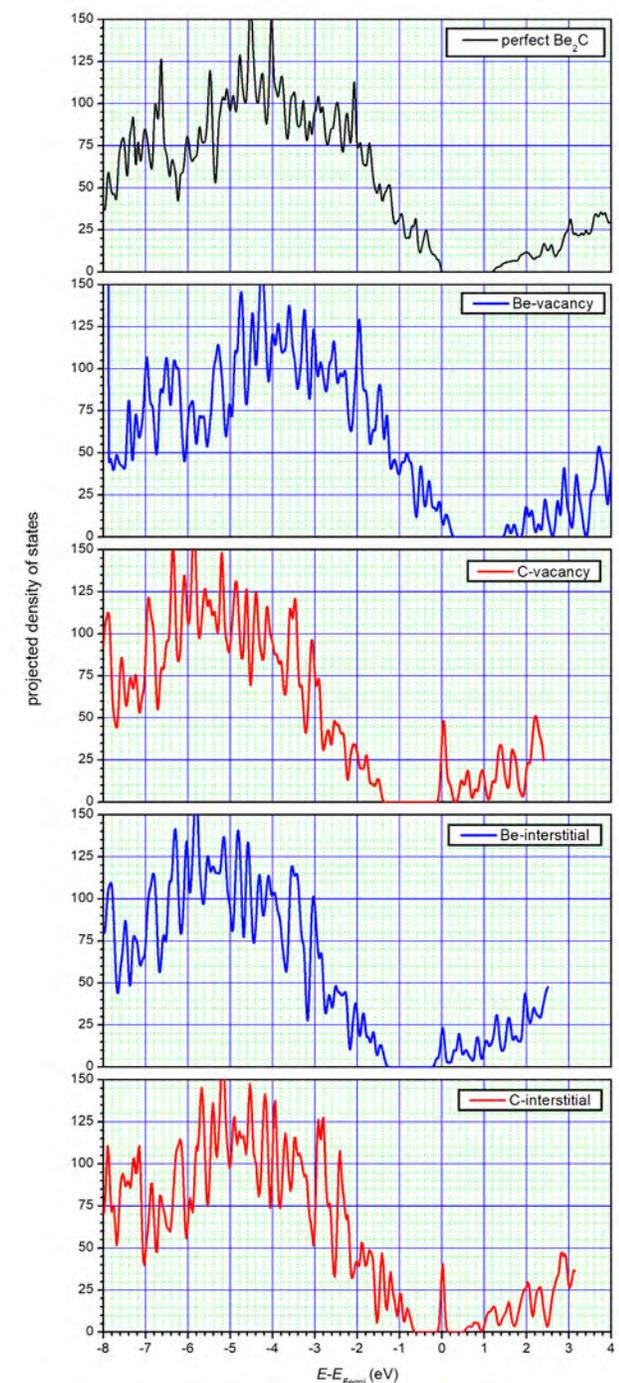
$$E_g = 1.212 \text{ eV}$$

- **Estimated Fermi energy:**

$$E_{Fermi} = 6.271 \text{ eV}$$

- **Defects change electronic structure by shifting energy and introducing new electronic states.**

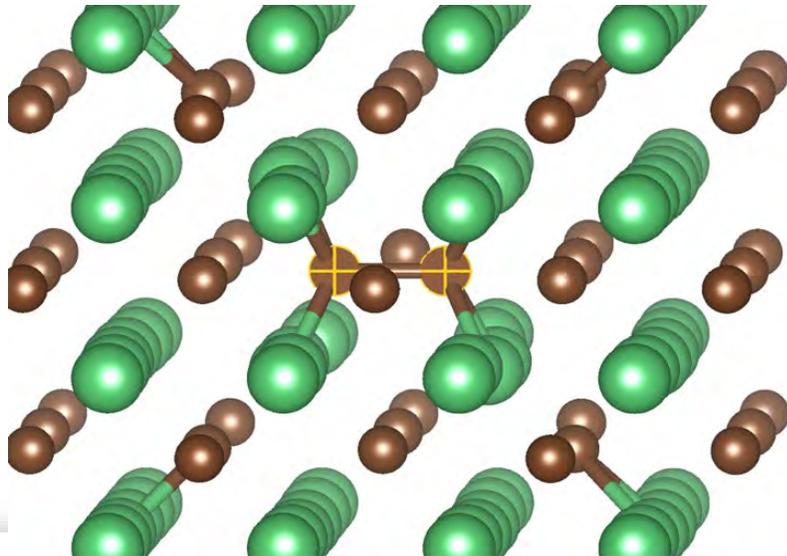
*Projected density of states in Be<sub>2</sub>C crystals top to bottom: perfect, and containing neutral Be-vacancy, C-vacancy, Be-interstitial and C-interstitial.*



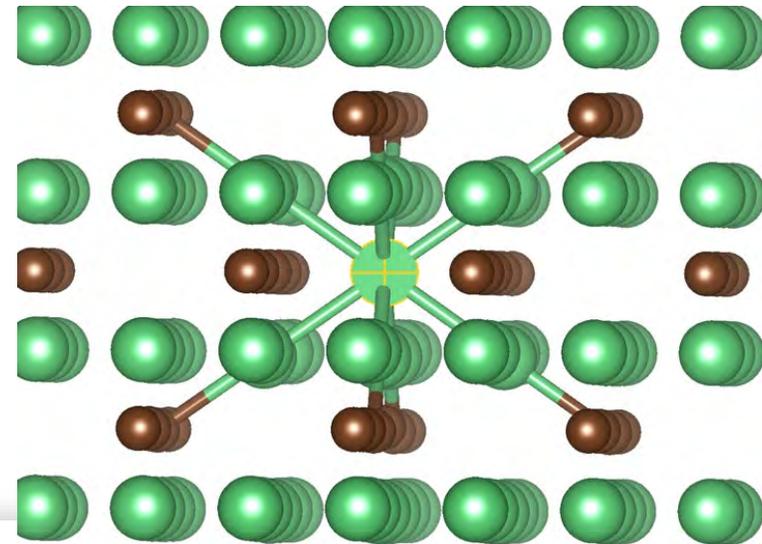
# Point defects properties interstitial structures

Ground state configurations:

- Anti-fluorite  $\text{Be}_2\text{C}$  structure assumes many possible configurations of interstitial atoms
- For estimating the ground state configuration, we applied DFT molecular dynamics modeling – annealing over 4 ps at 1200K followed by relaxation to 0K



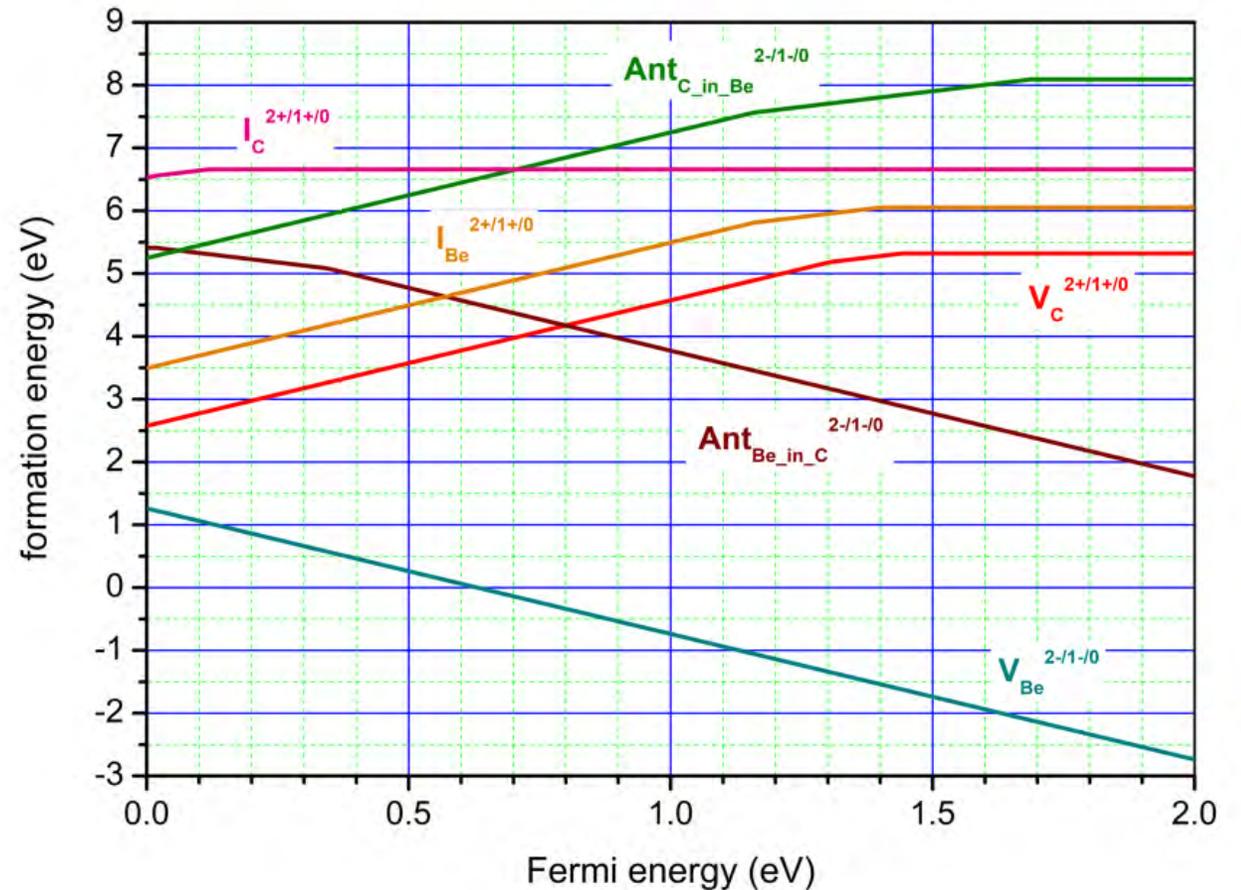
Symmetric split C-C dumbbell  
along [100] direction;



Octahedral Be-interstitial between  
the Be (001) planes.

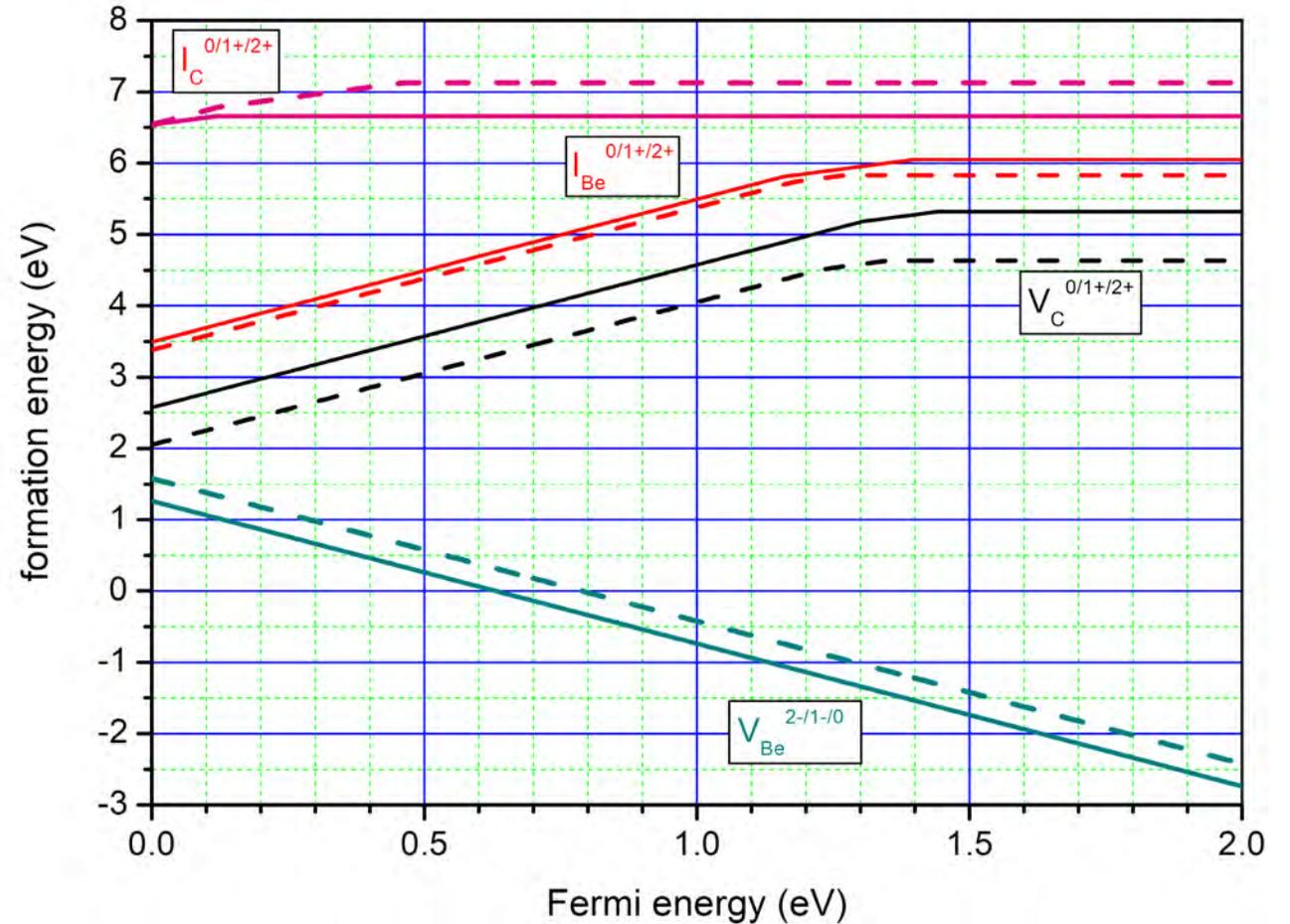
# Point defects properties – energy formation

- Formation energy of point defects strongly depends on their charge state;
- Defects presented in the plot :
  - $I_C$  – C-interstitial atom
  - $I_{Be}$  – Be-interstitial atom
  - $V_C$  – vacancy in C-site
  - $V_{Be}$  – vacancy in B-site
  - $Ant_{Be\_in\_C}$  – Be atom in C site
  - $Ant_{C\_in\_Be}$  – C atom in Be site



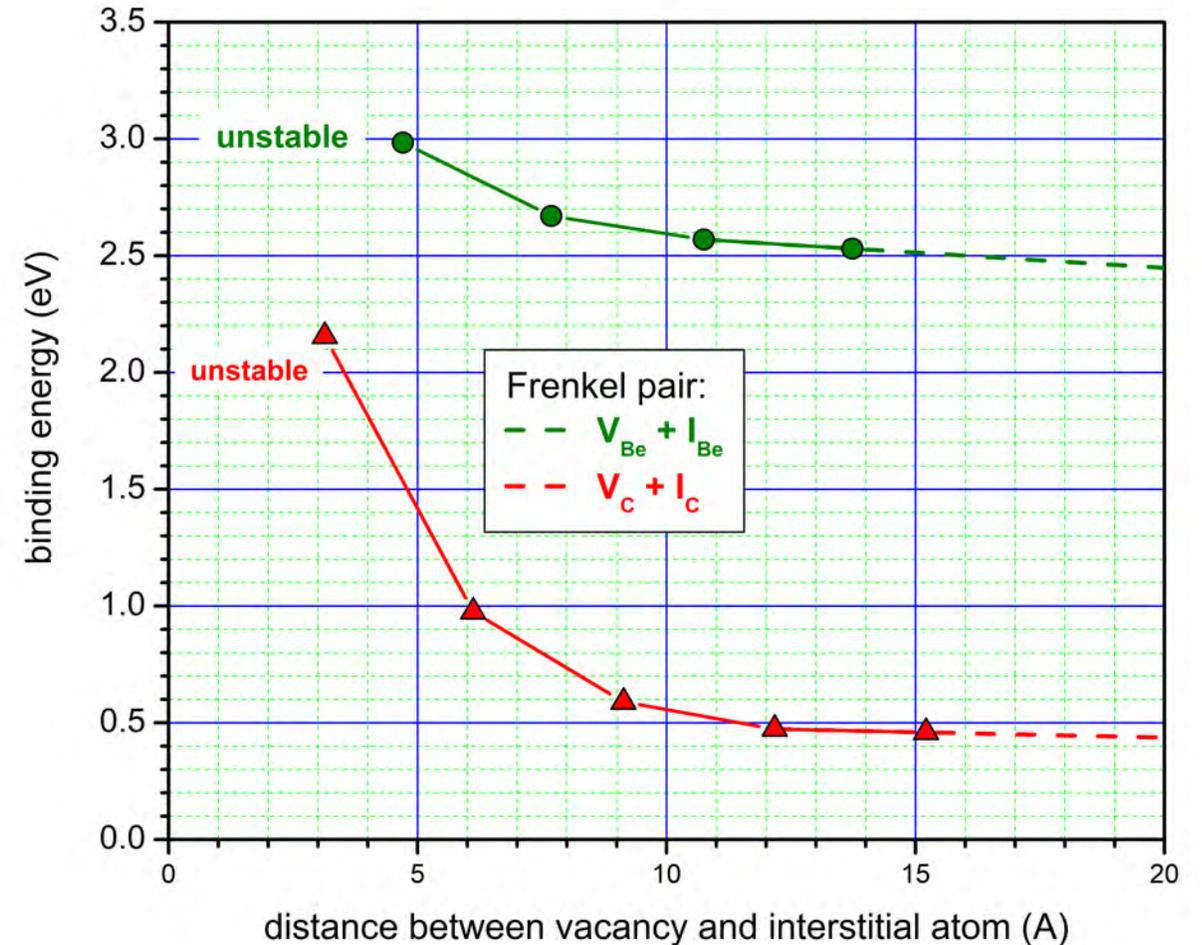
# Point defects properties - model size effect

Vacancy and interstitial atom formation energies in the smallest, i.e. 3x3x3, (solid lines) and largest, i.e. 5x5x5, (dashed lines) supercells.



# Point defects properties – Frenkel pairs

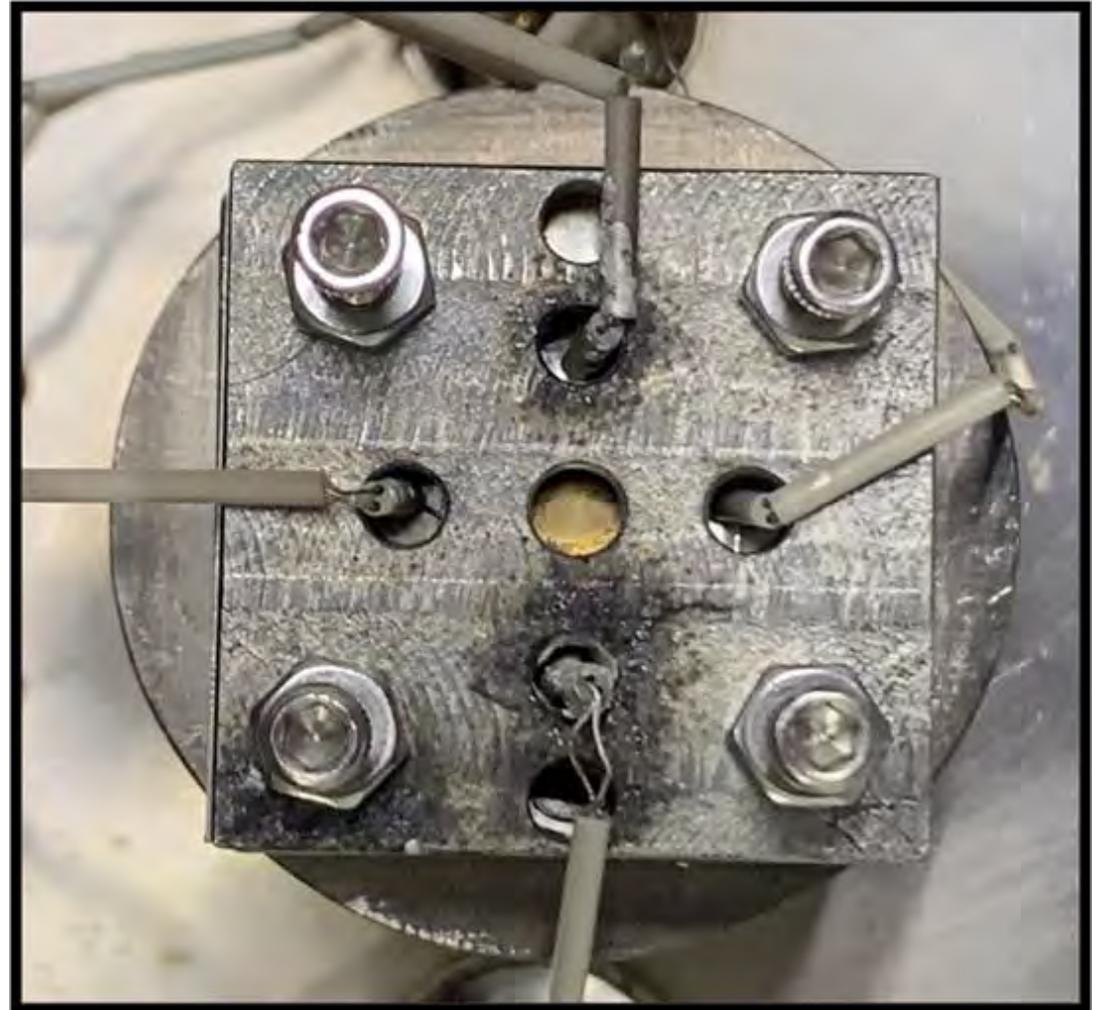
- Be and C FP modelled in the large supercell 5x5x5 (1500 sites)
- Vacancy and interstitial atoms were separated by different distance along close to  $\langle 111 \rangle$  direction.
  - Pair in the first coordination spheres were unstable.
- Binding energy was calculated relatively the neutral point defects:
  - Reasonable for C-FP when energy drops to  $\sim 0.5$  eV (instead of 0 eV)
  - Unlikely for Be-FP where energy saturates at  $\sim 2.5$  eV



# Ion Irradiation of $\text{Be}_2\text{C}$

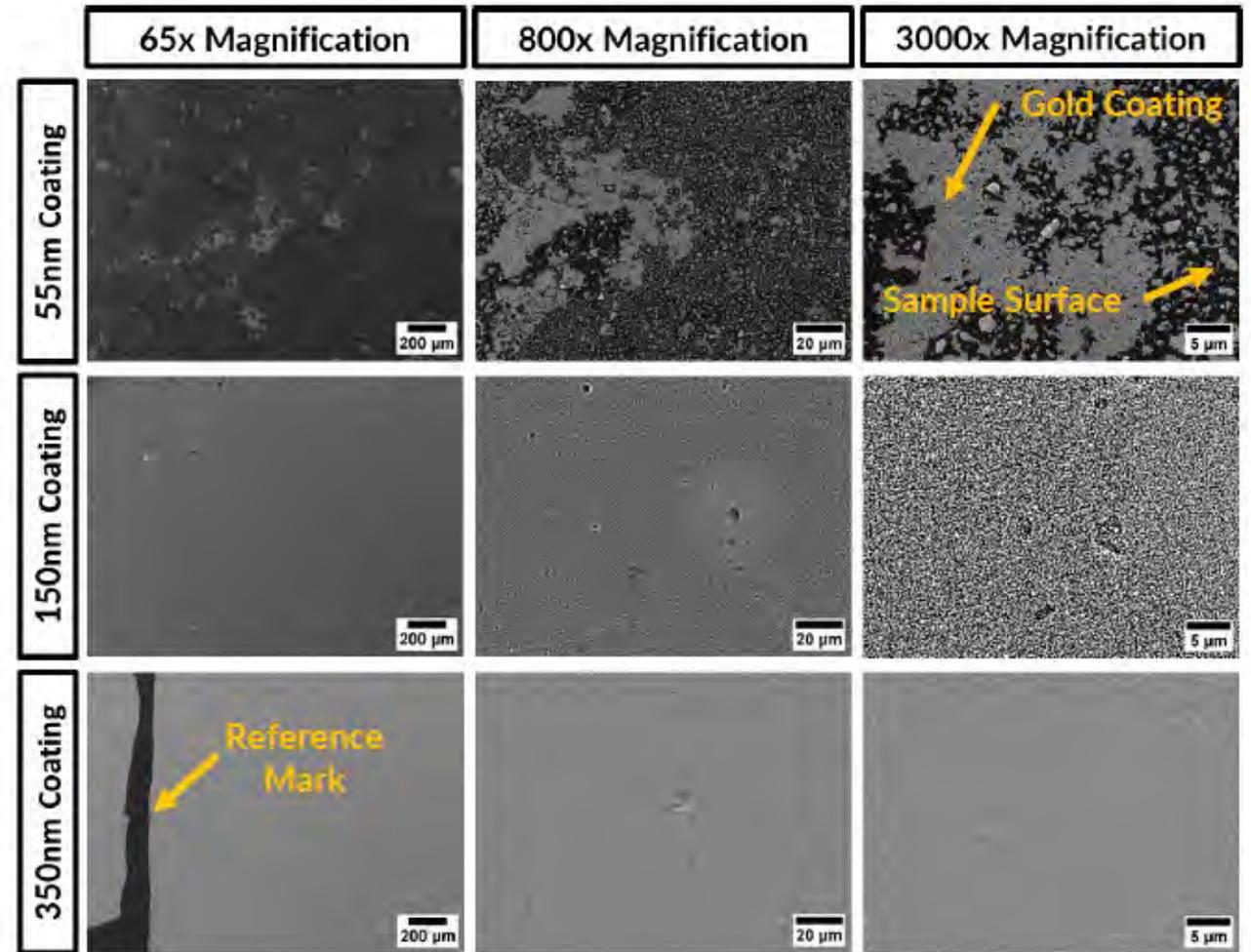
Diego Múzquiz

Stephen Raiman



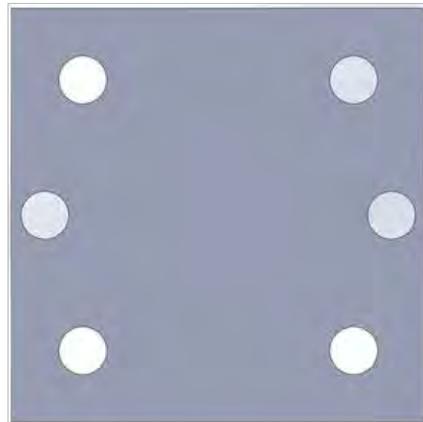
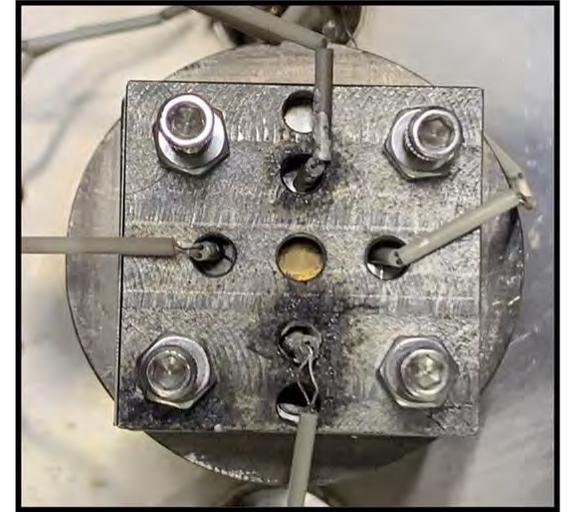
# Primary Containment for Irradiations

- Irradiation parameters minimize sputter yield to an acceptable limit
- Gold Layer further reduces sample sputtering
- 3 different coating were tested on SiC to support simulated results

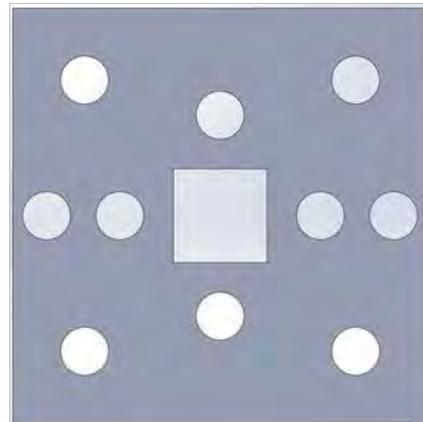


# Secondary Containment Failsafe

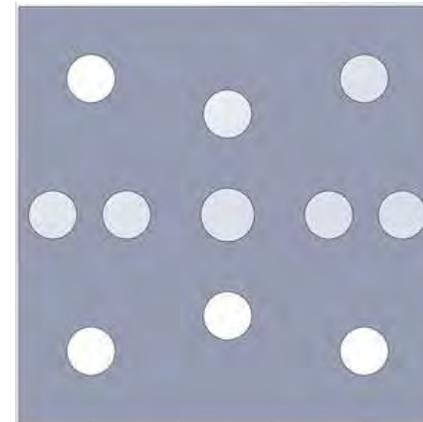
- **Sample inside custom molybdenum box attaches to stage**
- **Molybdenum has high heat transfer while not melting at experimental temperatures**



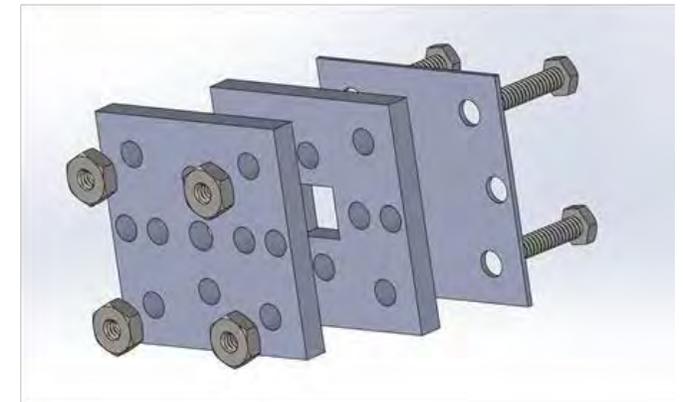
Thermal plate



Holding plate

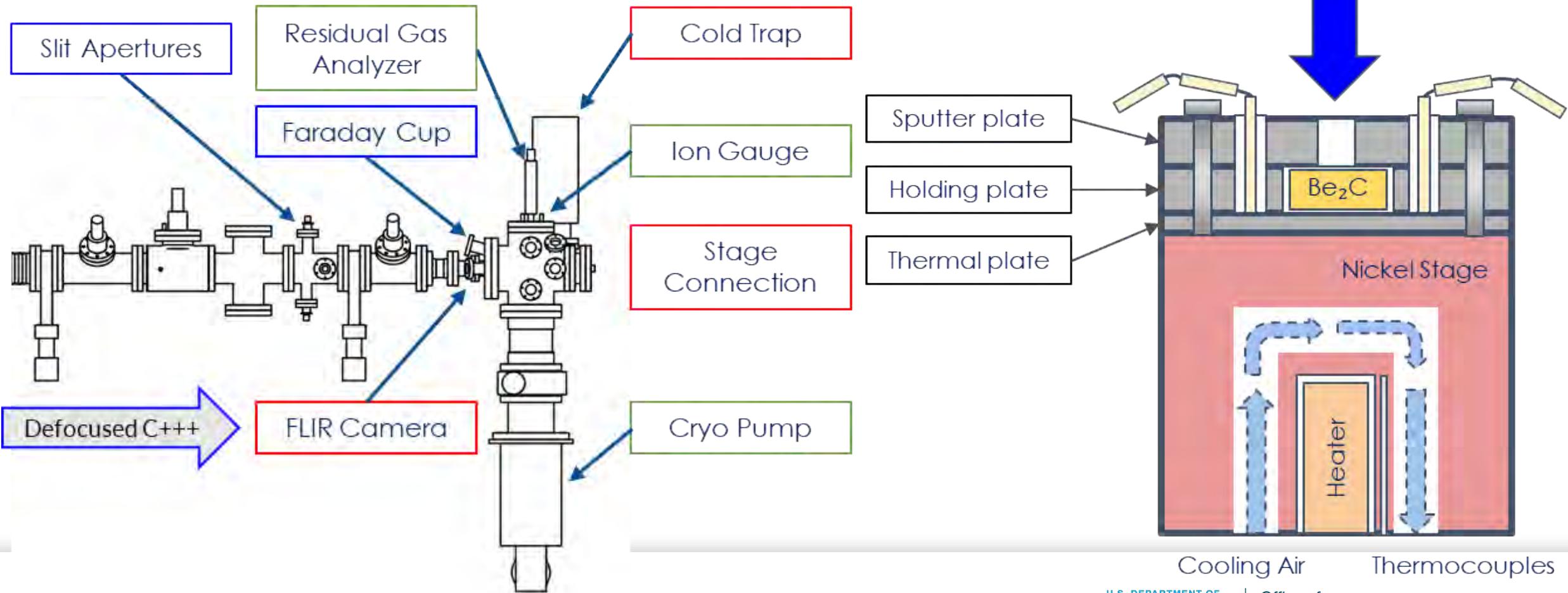


Sputter plate



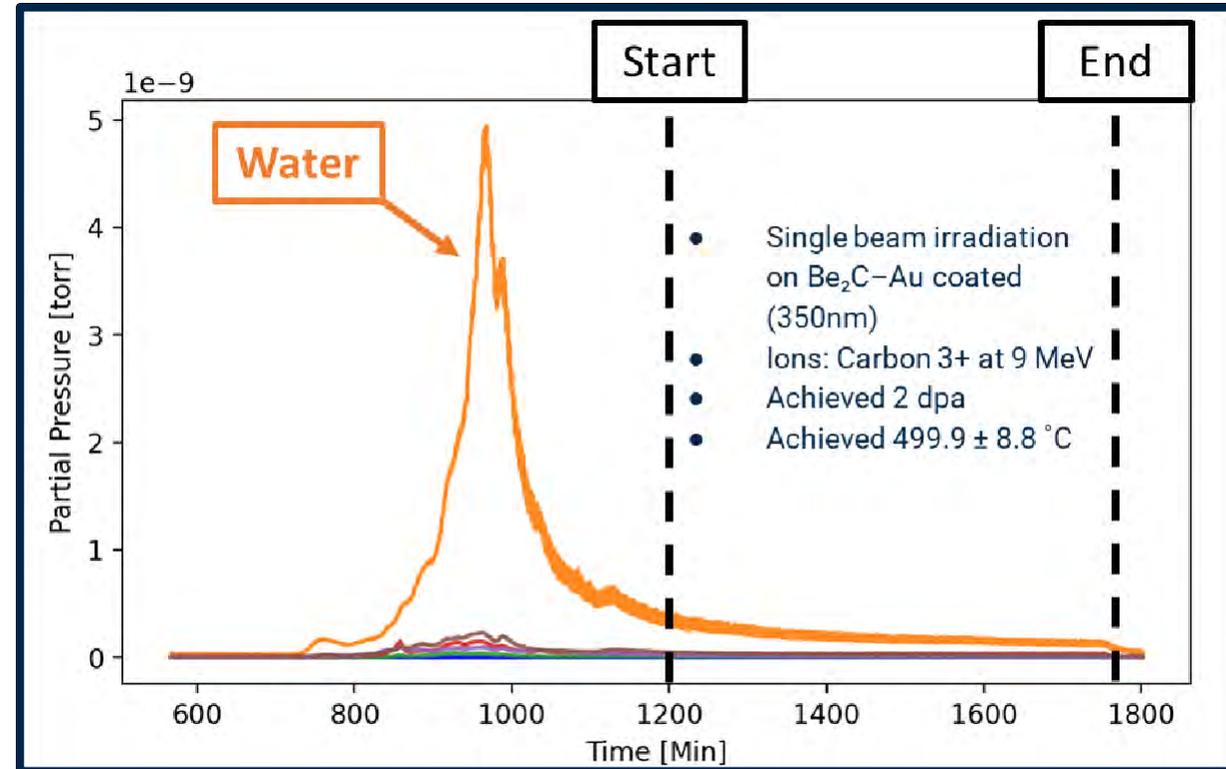
Assembly

# Irradiation Experimental Setup



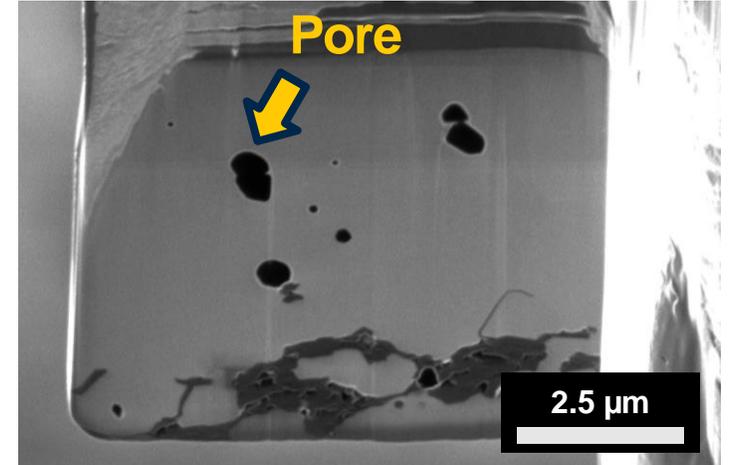
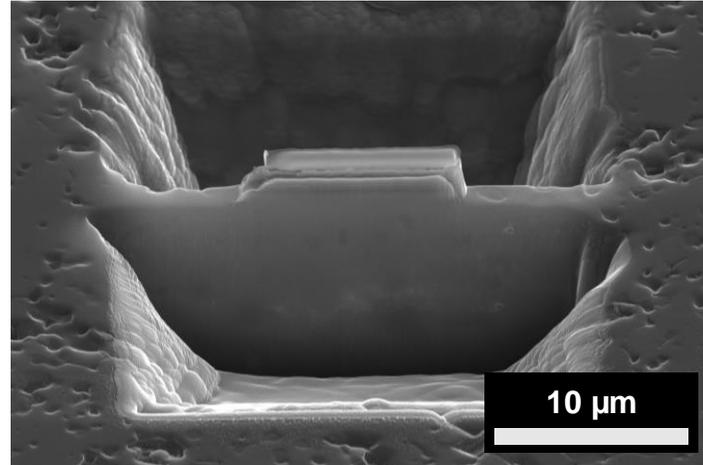
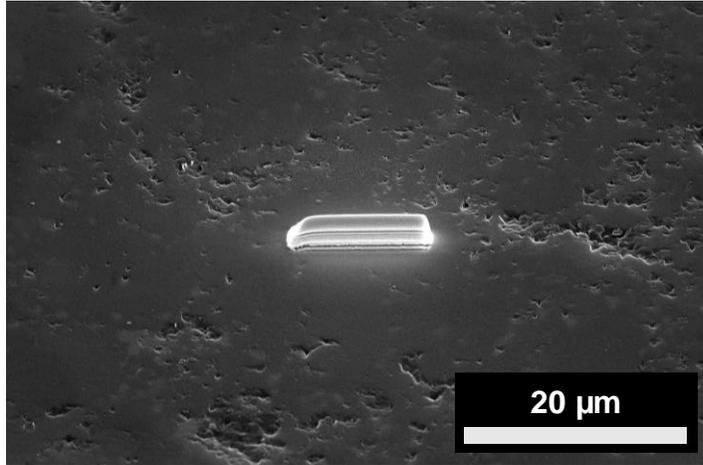
# Containment Monitoring and Swab Testing Post Irradiation

- **The RGA monitors the inside of the chamber**
- **Post irradiation, cleaning with a HEPA rated vacuum**
- **Swab tests are done post cleaning to ensure no beryllium remains**

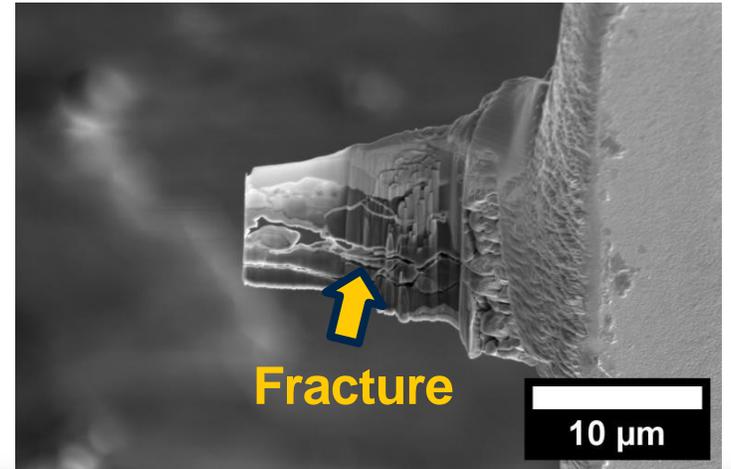
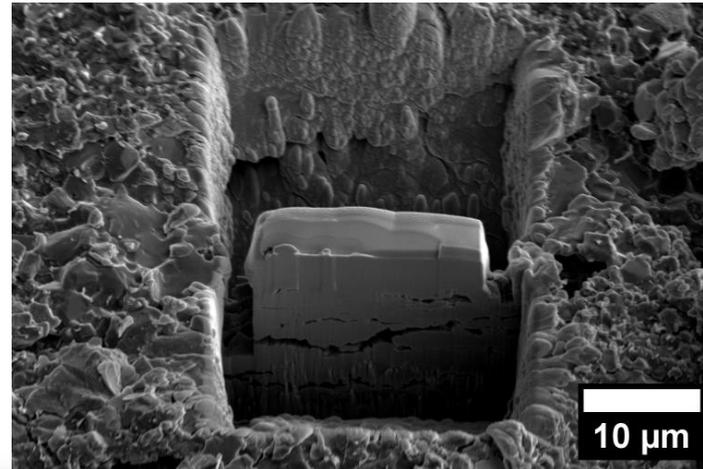
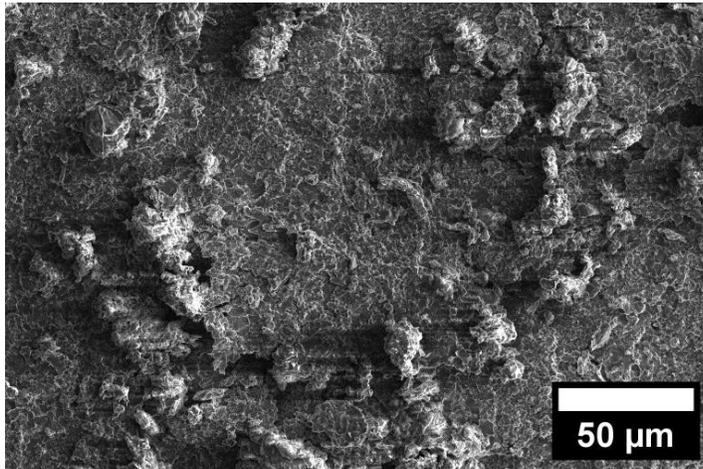


# Scanning Electron Microscopy

Pre-Irradiated



Post-Irradiated



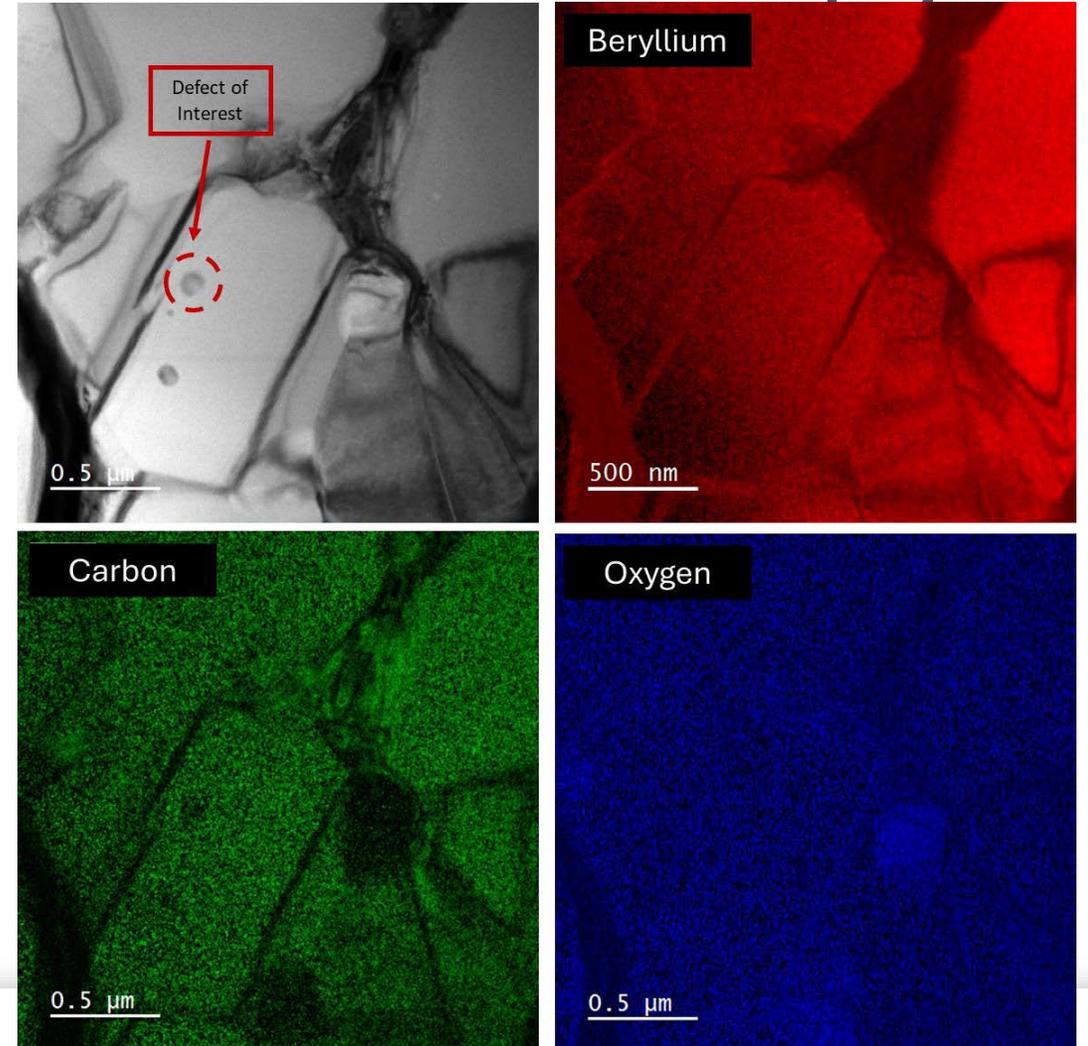
Surface

Trench Section

Clean Cross Section

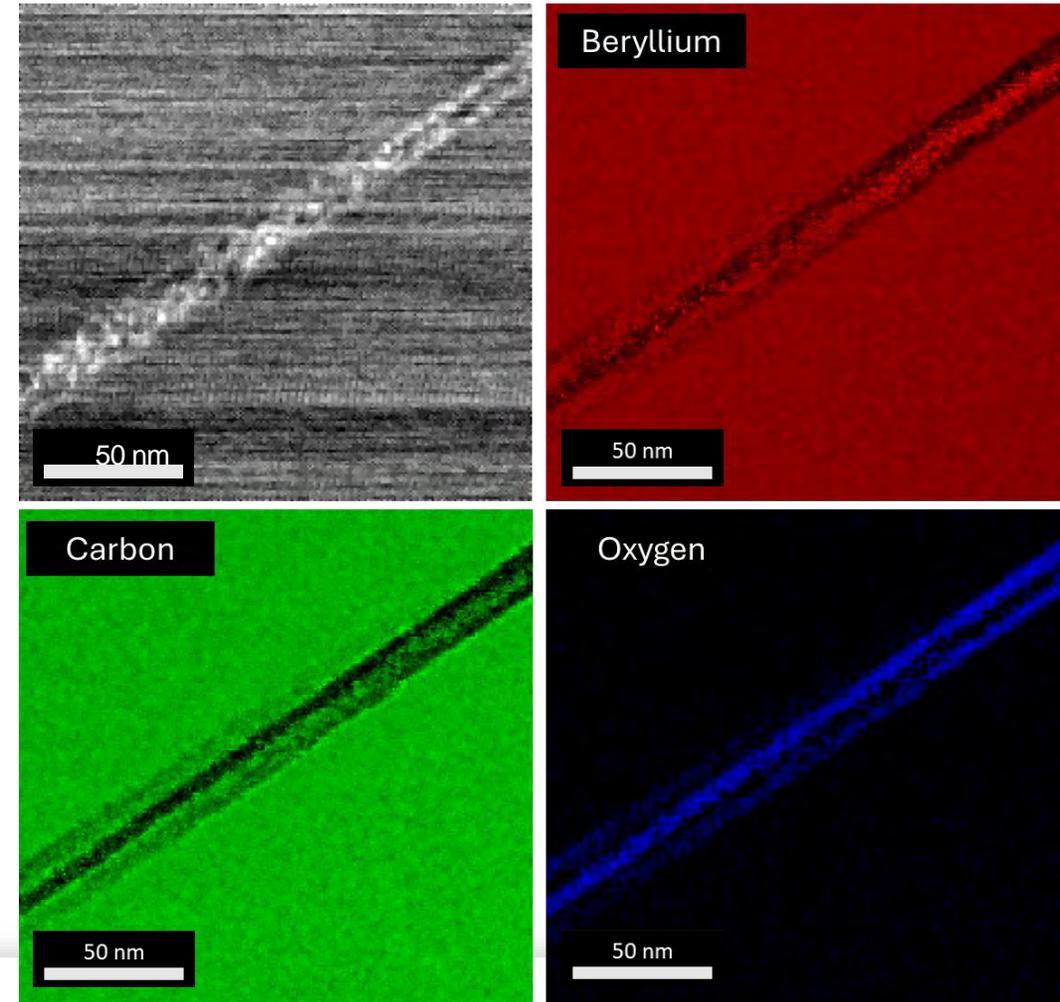
# Transmission Electron Microscopy

- 5-month gap between experiment and examination
- Carbon depleted regions
- Oxygen rich areas

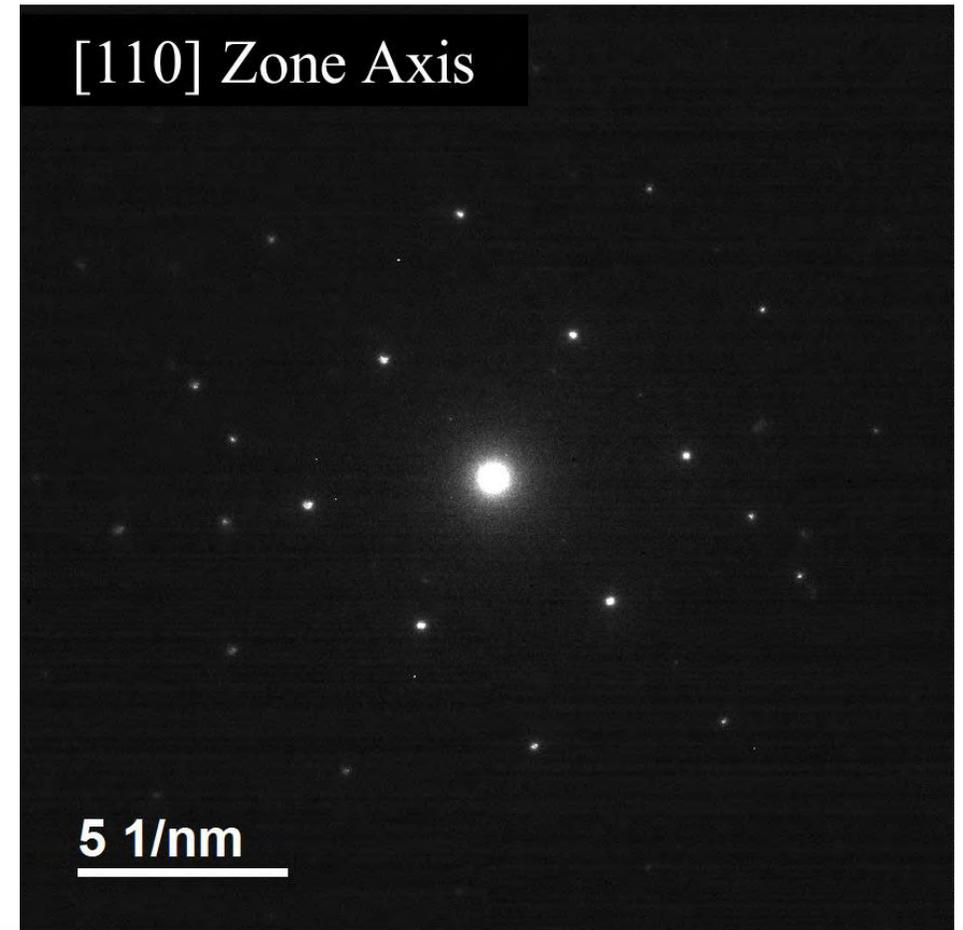
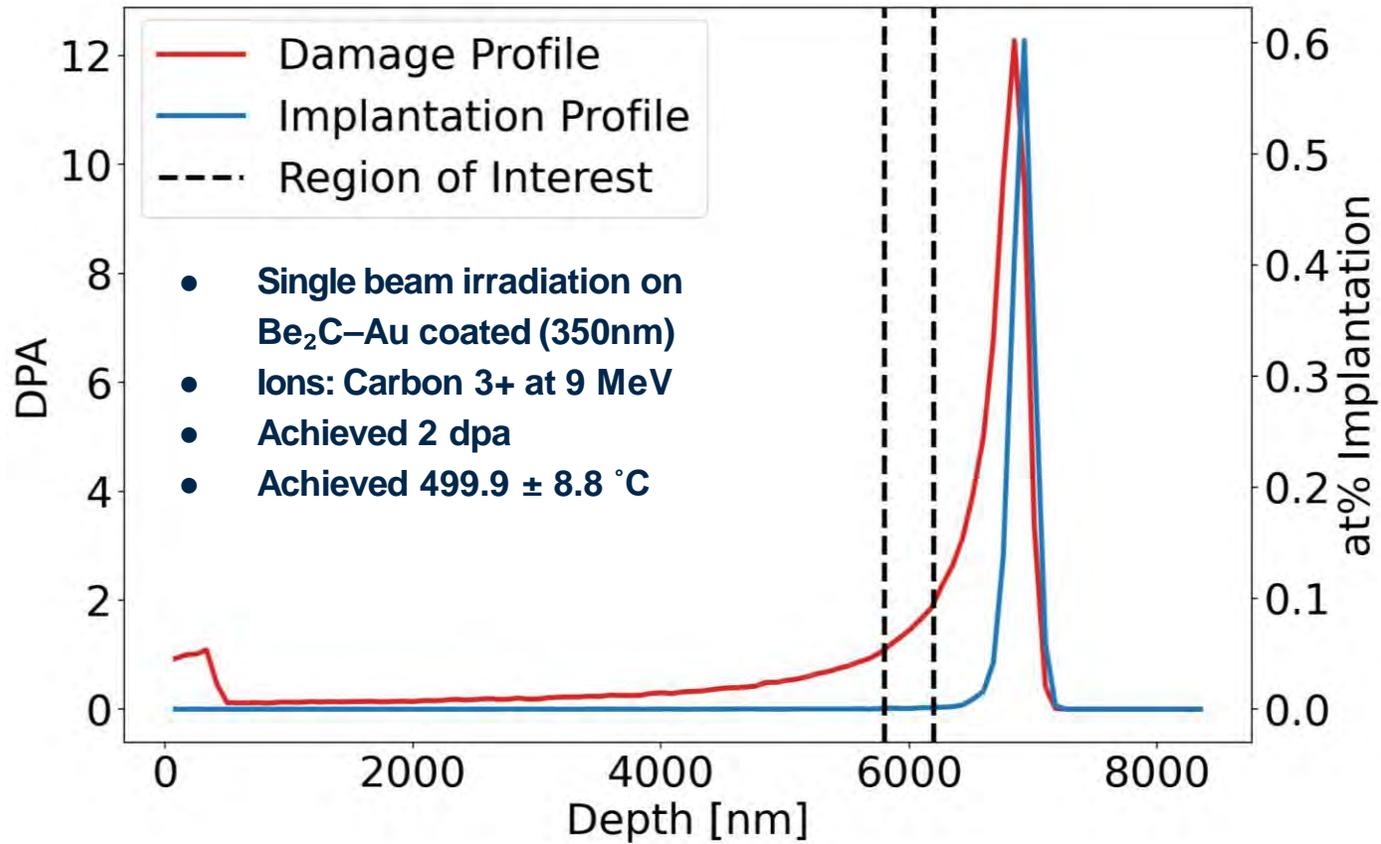


# Crack Propagation from Oxidation

- $Be_2C + H_2O \rightarrow 2BeO + CH_4$
- Methane form in exposed area
- Stress applied initiates cracks
- New surface area exposed
- Methane forms from new exposed area



# No Amorphization was Observed



# Upcoming Work

A. Willoughby

E. Cakmak

K. Johnson

B. Henry

E. Paxton

S. Fiscor

# High Temperature Stability

- **Already tested 650°C for 1 day, 1 week, and 2 weeks**
- **7 samples prepared and being measured via XRD for next set of exposures**
  - 600°C: 1 day, 1 week, 2 weeks, 1 month
  - 700°C: 1 day, 1 week, 2 weeks



Sample mounted on Mo capsule lid (courtesy J. Keiser)



Inner and outer capsules (courtesy J. Keiser)

# Degradation in hydrogen environment

- **Be<sub>2</sub>C degrades to methane in the presence of hydrogen**
  - Can this be used for <sup>3</sup>H mitigation?
- **Small pieces of Be<sub>2</sub>C will be exposed to controlled gas environments at 650°C (100% Ar, Ar + 1%H, Ar + 4%H)**
  - Quantify the rate of decomposition of Be<sub>2</sub>C
  - Critical knowledge for future use in MSR where hydrogen and tritium will be produced both by neutron capture in the solid Be and Be containing salts



# Suggested future modeling activity

**Understanding mechanisms of radiation damage and microstructure evolution assumes the following modeling activity:**

- **Predicting diffusion:**

- ✓ vacancy – vacancy jump barriers and kinetic Monte Carlo modeling vacancy diffusion;
- ✓ interstitial atoms – because of the complexity of diffusion mechanisms, direct molecular dynamics modeling should be applied;

- **Defect-defect interactions:**

- ✓ dilatation properties of vacancy and interstitial defects needed for long-range interaction in strain fields;
- ✓ extended defects nucleation and growth mechanisms and energy and structure properties;
- ✓ charge effects in defect-defect interactions;

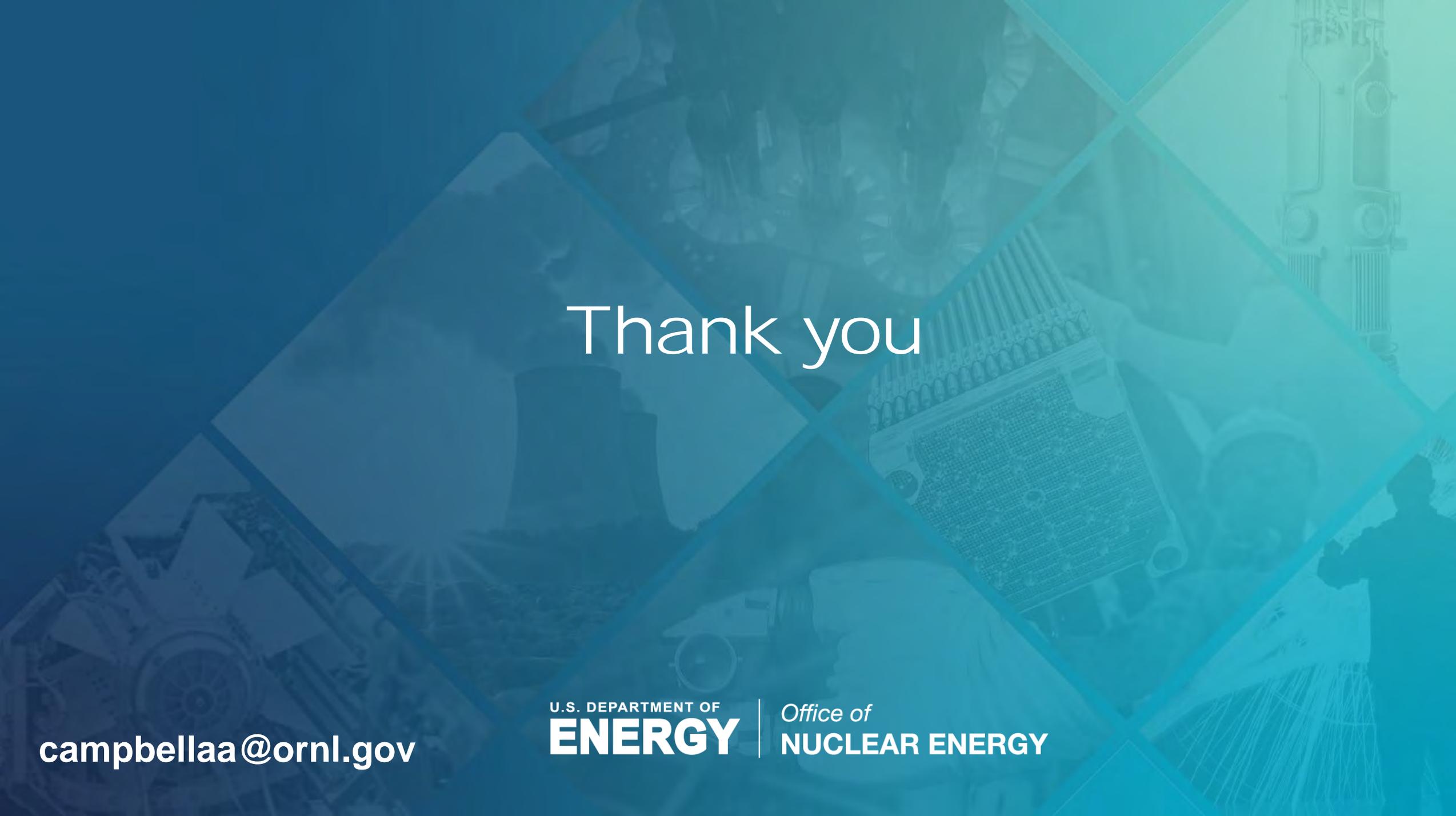
- **Development of kinetic Monte Carlo model for the overall dynamics of microstructure evolution.**

# Additional Ion Irradiation Studies

- **30 dpa (going on right now)**
- **Use ion implantation to study H and He diffusion characteristics**
- **In-Situ dual-beam irradiation (C<sup>+</sup> with simultaneous H implant)**
- **Static and flowing FLiBe exposures**

# Beyond FY24

- **From these preliminary results evaluate if a neutron irradiation campaign is viable. Discussing with AMMT about this work as it now is part of the AMMT portfolio going forward**
- **Work with Materion to develop advanced processing methods to tailor material properties**
- **Any future work will require setup of capabilities for handling and testing solid Be<sub>2</sub>C both pre- and post-irradiation**
  - Glove boxes, testing equipment (mass/dimensions, elastic properties, strength, CTE, thermal diffusivity, etc.)



Thank you

[campbellaa@ornl.gov](mailto:campbellaa@ornl.gov)

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